



William J. Cahill, Jr.
Chief Nuclear Officer

April 26, 1996
JPN-96-019

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

**Subject: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Extension of the Second Ten-Year Inservice Inspection and
Inservice Testing (ISI/IST) Intervals.**

- References:
1. NYPA letter, W.A. Josiger to NRC (JPN-94-037), "Second Ten-Year ISI/IST Interval Extension," dated August 2, 1994.
 2. NUREG-1482, "Guidelines for Inservice Testing at Nuclear Power Plants," dated April 1995.
 3. NRC letter, J.E. Menning to W.J. Cahill, Jr., "Extension of Second 10-Year Inservice Inspection (ISI) and Inservice Testing (IST) Interval - James A. FitzPatrick Nuclear Power Plant (TAC No. M90033)," dated October 26, 1994.
 - 4) NYPA letter, W.J. Cahill, Jr to NRC (JPN-95-005), "Reactor Vessel Head Weld Flaw Indication Inspections and Evaluation Analysis," dated February 9, 1995.

Dear Sir:

This letter is submitted to inform the NRC that the Authority is extending the FitzPatrick second ten-year Inservice Inspection and Inservice Testing (ISI/IST) interval by a period of nine (9) months. The FitzPatrick plant is currently developing a risk-informed ISI pilot program for submittal to the NRC. This interval extension will allow additional time for the Authority to develop the risk-informed ISI Program, and will eliminate the need to submit two separate ISI programs for NRC review. The extension is being applied to the third period of the second interval currently scheduled to end September 1996. The additional nine months will extend this interval through June 1997.

The ASME Section XI code applicable to FitzPatrick (1980 Edition with Addenda through Winter 1981), Paragraph IWA-2400(c), permits each inspection interval to be decreased or extended by as much as one year, and allows intervals to be extended when outages greater than six months occur. The Authority previously extended the second ISI/IST

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Interval by a period of 14 months (Reference 1) to account for the length of the 1992 refueling/maintenance outage. This extension of nine months is acceptable as demonstrated by ASME Committee Interpretation XI-1-86-54 which stated that the one year extension may be applied in addition to an extension taken to account for outages of greater than six months in duration. In addition, ASME Interpretation XI-1-92-57 indicated that the Code allows an inspection interval to be extended or decreased for reasons other than to enable an inspection to coincide with a plant outage. These positions are consistent with the NRC guidance stated in Reference 2, Section 3.3.1.

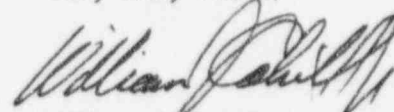
The NRC staff stated (Reference 3) that, based on the original dates of the second 10-year interval, the Authority is required to complete the augmented reactor vessel examinations required by 10 CFR 50.55a(g)(6)(ii)(a) within one 40 month period after July 1995 (i.e., by November 1998). This commitment is not affected by the nine month extension. This extension does not affect performance of IGSCC, erosion / corrosion, in-vessel inspection programs, or inspection of the reactor vessel head flaw (Reference 4) during the upcoming refueling outage.

The Authority is making no new commitments in this submittal.

The Authority's Authorized Nuclear Inservice Inspector (ANII) has reviewed the proposed interval extension and has no concerns.

If you have any questions, please contact Mr. A. Zaremba.

Very Truly Yours,



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cc: Regional Administrator
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