NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Cooper Nuclear Station	0 5 0 0 0 2 9 8	1 OF 0 2

TITLE (4)
High Pressure Coolant Injection Overspeed Trip Control Valve Diaphragm Failure

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											0 5 0 0 0
0 8	2 8	8 4	8 4	—	0 1 1	— 0 1	1 0	3 1	8 4		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.406(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
0 8 9		20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
E. M. Mace, Plant Engineering Supervisor	4 0 2 — 8 2 5 - 3 8 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
E	B J	P C V	R 2 9 0	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Revision 0 of LER 84-011 reported the failure of the High Pressure Coolant Injection (HPCI) overspeed trip control valve diaphragm. The text of this revision (Rev. 1) discusses information related to this event which was discovered after the submittal of Revision 0.

IE22 1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Additional information was discovered which indicated that the failed diaphragm was not the original diaphragm supplied with the unit, as originally reported. The failed diaphragm was a replacement diaphragm installed on March 3, 1982. This improved design diaphragm was installed per General Electric Service Information Letter (SIL) 358, dated June 1981. This SIL addressed a problem with procurement of valve diaphragms for the subject control valve application. This improved version of the diaphragm is designed for a minimum of 1000 cycles. Plant operating records indicate that the failed diaphragm was cycled approximately 950 times before failure, between March 3, 1982 and August 28, 1984.

The preventative maintenance instituted as corrective action to prevent recurrence per Revision 0 of this LER has been reevaluated. As a result, these preventative maintenance requirements are considered to be adequate. No further corrective action is planned.

Nebraska Public Power District

COOPER NUCLEAR STATION
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321
TELEPHONE (402) 825-3811

CNSS840410

October 31, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Revision 1 to Cooper Nuclear Station Licensee Event Report 84-011 is forwarded as an attachment to this letter.

Sincerely,

P. V. Thomason
Division Manager of
Nuclear Operations

PVT:lb
Attach.
cc: R. D. Martin
L. G. Kuncl
J. D. Weaver
L. R. Berry
INPO Records Center
ANI Library

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