

OPERATING DATA REPORT

DOCKET NO: 50-368
 DATE: October 1984
 COMPLETED BY: L.S. Bramlett
 TELEPHONE: 501-964-3145

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: September 1-30, 1984
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

	MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	39,599.0
12. Number of Hours Reactor was Critical	658.8	5,535.7	27,162.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,430.1
14. Hours Generator On-Line	654.3	5,369.5	26,319.8
15. Unit Reserve Shutdown Hours ..	0.0	0.0	75.0
16. Gross Thermal Energy Generated (MWH)	1,715,685.0	13,839,425.0	66,388,965.0
17. Gross Electrical Energy Generated (MWH)	570,822.0	4,606,135.0	21,623,086.0
18. Net Electrical Energy Generated (MWH)	544,498.0	4,394,133.0	20,600,473.0
19. Unit Service Factor	90.9	81.7	66.5
20. Unit Availability Factor	90.9	81.7	66.7
21. Unit Capacity Factor (Using MDC Net)	88.1	77.9	60.6
22. Unit Capacity Factor (Using DER Net)	82.9	73.3	57.0
23. Unit Forced Outage Rate	9.1	7.8	18.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>None</u>			
25. If Shut Down At End of Report Period. Estimated Date of Startup: _____			
26. Units in Test Status (Prior to Commercial Operation): _____			

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368
 UNIT: Two
 DATE: October 1984
 COMPLETED BY: L.S. Bramlett
 TELEPHONE: 501-964-3145

MONTH September 1984

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	0
2	0
3	33
4	221
5	281
6	796
7	882
8	882
9	887
10	882
11	889
12	889
13	890
14	890
15	901
16	904
17	902
18	900
19	899
20	899
21	898
22	894
23	890
24	838
25	865
26	904
27	903
28	902
29	904
30	862

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

SEPTEMBER 1984

UNIT 2

The unit began the month shutdown in order to repair a RCP seal leak and a steam generator manway leak. The unit was placed on line on September 3 and reached 100% full power (FP) on September 7. On September 24, power was reduced to 75% to repair an EH fluid leak on the "B" MFW pump EHC system. The unit was returned to 100% power on September 25 and remained there through September 30, when power was reduced to 75% because of secondary chemistry problems.

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT FOR SEPTEMBER 1984

DOCKET NO	50-368
UNIT NAME	ANO-2
DATE	10/01/84
COMPLETED BY	L.S. Bramlett
TELEPHONE	501-964-3145

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
8408	840828	F	65.7	A	4	2-84248	ZZ	ZZZZZZ	The unit tripped due to a dropped CEA. The unit then went to CSD to repair a faulty RCP seal and a leaking steam generator manway.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training &
License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation
5-Load Reduction
9-Other

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)
5
Exhibit 1 - Same Source

DATE: September 1984

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. April 1985
3. Scheduled date for restart following refueling. July 1985
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Yes, some software changes to the Core Protection Calculators will be made, and this will cause some technical specification changes.
5. Scheduled date(s) for submitting proposed licensing action and supporting information. February 1985
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Burnable poison rods will be used in reload fuel.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 168
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

present 988 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 2003



ARKANSAS POWER & LIGHT COMPANY

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October 15, 1984

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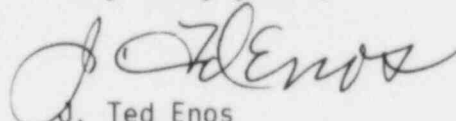
Mr. Harold S. Bassett, Director
Division of Data Automation
and Management Information
Office of Resource Management
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Monthly Operating Report
(File: 2-0520.1)

Gentlemen:

Attached is the NRC Monthly Operating Report for September 1984 for Arkansas Nuclear One - Unit 2.

Very truly yours,


J. Ted Enos
Manager, Licensing

JTE:SAB:ac

Attachment

cc: Mr. Robert D. Martin
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555