

**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) EDWIN I. HATCH, UNIT II										DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 1 OF 0 2										PAGE (3) 1		
TITLE (4) ESF Actuation Due To RPS "B" MG Set Trip																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)							
0	1	2	1	8	5	8	5	0	0	4	0	0	2	1	9	8	5	0	5	0	0	0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																			
2			20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)			20.405(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(c)							
0 1 1 3			20.405(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 356A)							
			20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
			20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																						
NAME												TELEPHONE NUMBER										
T. L. Elton, Acting Superintendent of Regulatory Compliance												9 1 2 3 6 7 1 7 8 5 1 1										
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS												
x	JIC	IRIGG	10810	y																		
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR						
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO										
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																						
<p>On 01/21/85, with the reactor mode switch in the startup and hot standby position and reactor power at 314 MWt (approximately 13% power), and on 01/23/85, with the reactor mode switch in the run position at 1549 MWt (approximately 64% power), the Reactor Water Clean-up (RWCU) outboard isolation valve (2G31-F004) isolated due to a 1/2 group isolation from channel "B" of RPS.</p> <p>These events were the result of "B" RPS MG Set tripping due to a failed voltage regulator.</p> <p>The failed voltage regulator was replaced and adjusted to approximately 120 volts. The regulator was then verified to regulate (i.e., remain at 120 volts plus or minus 2 volts) under varying loads per the manufacturer's recommendation. The MG Set was functionally tested satisfactorily and returned to service on 01/26/85.</p>																						
8502270669 850219 PDR ADOCK 05000366 S PDR IE 20 111																						

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2)  0 5 0 0 0 3 6 6 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		— 0 0	4	— 0	0 0	2	OF 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

This 30 day LER is required by 10CFR50.73(a)(2)(iv) since the Reactor Water Clean-up (RWCU) outboard isolation valve (2G31-F004) is a primary containment isolation valve and its isolation constitutes the actuation of an Engineered Safety Feature (ESF).

On 01/21/85, with the reactor mode switch in the startup and hot standby position and reactor power at 314 MWt (approximately 13% power), the RWCU outboard isolation valve (2G31-F004) isolated due to a 1/2 group isolation from channel "B" of RPS. This event is the result of "B" RPS MG Set's tripping due to an undervoltage on bus "B".

An investigation revealed that the MG Set's voltage regulator had drifted low to approximately 114 volts. The voltage regulator was readjusted to approximately 120 volts, and "B" MG Set was returned to service on 01/23/85. However, on 01/23/85 when the MG Set was returned to service the voltage regulator began to drift up to approximately 130 volts, thus the MG Set tripped and RWCU isolation valve 2G31-F004 isolated.

The voltage regulator was replaced and adjusted to approximately 120 volts. The regulator was then verified to regulate (i.e., remain at 120 volts plus or minus 2 volts) under varying loads per the manufacturer's recommendation. The MG Set was functionally tested satisfactorily and returned to service on 01/26/85.

No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 2 system or on Unit 1. The health and safety of the public were not affected by this non-repetitive event.

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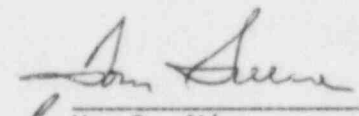
Edwin I. Hatch Nuclear Plant

February 19, 1985  
GM-85-130

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-004. This report is required by 10CFR 50.73(a)(2)(iv).

  
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H. C. Nix  
General Manager

HCN/TLE/vlz

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IE22  
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