NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Indian Point, Unit 3	0 5 0 0 0 2 8 6	1 OF 0 2

TITLE (4)

Steam Generator High Level Trips

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 1	2 3	8 5	8 5	0 0 2	0 0	0 2	2 1	8 5		0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10)	0 1 5	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
John Anderson	AREA CODE 9 1 4 — 7 3 9 - 8 2 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S J	F C V	B 0 4 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 23, 1985, a turbine trip and subsequent reactor trip were initiated by a high level in No. 31 Steam Generator. Reactor power was 15 percent and being increased at the time of trip. The high level was the result of a feedwater system perturbation initiated by the starting of a second condensate pump. The unit had been returned to criticality and was at 29 percent reactor power and increasing when a second high level trip in No. 31 Steam Generator occurred. This trip also occurred after a feedwater system perturbation that began when the recirculation valve for No. 31 Main Boiler Feed Pump was closed. Delayed response was noted in the operation of No. 31 Main Feedwater Regulating Valve after the second transient, and was also determined to be a contributing factor in the first trip. The valve was repaired and calibrated, and the unit was subsequently synchronized to the bus on January 23, 1985 and returned to full power.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Indian Point, Unit 3	0 5 0 0 0 2 8 6	8 5	— 0 0 2	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 0854 hours on January 23, 1985, a turbine trip and subsequent reactor trip were initiated by a high level in No. 31 Steam Generator (SG). Reactor power was 15 percent and increasing at the time of the trip. The high level resulted from a feedwater system transient which began after starting a second condensate pump. Attempts to check the ultimate level swell were unsuccessful. No. 31 Steam Generator was the first to reach the high level trip setpoint. After determining that no apparent equipment failures were involved, the unit was returned to criticality, synchronized to the bus at 1312 hours, and power ascension continued.

At 1501 hours on January 23, 1985, with reactor power at 29 percent and increasing, a turbine trip and subsequent reactor trip were initiated by a high level in No. 31 SG. This trip also occurred after a feedwater system transient. Following the closing of the recirculation valve on No. 31 Main Boiler Feed Pump (MBFP), the steam generator levels increased, which prompted the operator to limit the level increase with manual operation of the feedwater regulators. During this transient, No. 31 SG level reached the high level setpoint. Subsequently, it was determined that No. 31 Feedwater Regulating Valve (FRV) responded sluggishly to its control input signals, thus magnifying a transient that was within the normal capability of the plant equipment to sustain for both plant trips.

No. 31 FRV was inspected and found to have a misaligned positioner linkage assembly. The assembly, made by Bailey Meter Co., was replaced, and the valve was calibrated. In addition, the other FRVs were inspected and stroked and were found to be operating properly. The reactor was then made critical and the unit was synchronized to the bus at 2350 hours on January 23, 1985.

All equipment associated with the reactor trips operated correctly. No similar events have been reported in an LER to date.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200

NewYork Power Authority

February 21, 1985
IP-FWG-386

Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 85-002-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(iv).

Very truly yours,

John C. Brons
Resident Manager

FWG/bam
Attachment

cc: Dr. Thomas Murley
Regional Administrator
Region 1
U. S. Nuclear Regulatory Commission
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