

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

BYRON, UNIT 1

DOCKET NUMBER (2)

0510000454 1 OF 02

PAGE (3)

TITLE (4)

FAILURE TO DE-ENERGIZE PORV BLOCK VALVES

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
01	19	85	85	017	00	02	19	85		0510000454	

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9)	20.402(b)	20.408(a)	50.73(a)(2)(iv)	73.71(b)
3	20.408(a)(1)(i)	50.38(a)(1)	50.73(a)(2)(v)	73.71(a)
POWER LEVEL (10) 000	20.408(a)(1)(ii)	50.38(a)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
	20.408(a)(1)(iii)	X 50.73(a)(2)(ii)	50.73(a)(2)(vii)(A)	
	20.408(a)(1)(iv)	50.73(a)(2)(iii)	50.73(a)(2)(vii)(B)	
	20.408(a)(1)(v)	50.73(a)(2)(iv)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

Carolyn Kilbride, System Test Engineer, Ext. 245

TELEPHONE NUMBER

AREA CODE

815 234-1544

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
A	AB	PZR	C635	N					

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

ABSTRACT Limit to 1400 words. If space is limited, use single-space typewritten text (16)

During the performance of Start Up Test 2.63.33, Reactor Coolant System Leak Test, both pressurizer Power Operated Relief Valves (PORVs), IRY455A and IRY456, and their associated block valves, IRY8000A and IRY8000B, had their control switches placed to the closed position without removing power from the block valves within the required one hour Tech Spec limit (Tech Spec 3.4.4.b). This was due to a misinterpretation of the applicable Tech Spec LCO by the operating personnel. To correct the situation, the test was interrupted, the valves were returned to operable status, and the operating personnel were briefed on the intent of the Tech Spec LCO. The Start Up Test was then re-entered and successfully completed in adherence to the Tech Specs.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	0 1 7	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 368A's) (17)

AT 0530 on 1-19-85 with Unit One in Mode 3, both pressurizer Power Operated Relief Valves (PORVs), 1RY455A and 1RY456, and their associated block valves, 1RY8000A and 1RY8000B, had their control switches placed to the "CLOSE" position under the direction of Start Up Test 2.63.33, Reactor Coolant System Leak Test. This was necessary in order to facilitate pressurization of the Reactor Coolant System (RCS) to the test pressure required which was above the PORV lift setpoint. It was recognized in the test procedure that placing the valve control switches in the CLOSE position, which closed the block valves and prevented automatic PORV operation, made the PORV's inoperable as defined in the Technical Specifications. A note in the test procedure expressed this concern by stating: "...the LCO for Tech Spec 3.4.4 will be intentionally exceeded. LCOAR 1BOS 4.4-1a will be in effect. Power may have to be removed from the PORV block valves."

However, there was misunderstanding as to the intent of the Tech Spec action statement. Power to the block valves was not removed within the one hour time limit because the operator wanted to maintain the availability of the PORV's for pressure relief should this be required.

The failure to satisfy the requirements of the LCO was recognized during the shift turnover, and actions were initiated to restore the PORV's to an OPERABLE alignment.

At 0719, the RCS was depressurized to normal operating pressure and the PORV's and their block valves were returned to operable status.

The corrective action taken was to brief the operating personnel on the intent of the Tech Spec LCO. The start up test was re-entered and successfully completed in adherence to the Tech Specs.

This event did not affect plant or public safety because failure to remove power from the block valves under the circumstances neither increased the probability of an accident nor of a radioactive release.

Previous occurrences - none.



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

DATE: February 19, 1985

LTR: BYRON 85-0261

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR 50.73(a)(2)(iv) which requires a 30 day written report.

This report is number 85-017-00, Docket No. 50-454.

Very truly yours,

R. E. Querio
Station Superintendent
Byron Nuclear Power Station

Enclosure: Licensee Event Report No. 85-017-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
CECO Distribution List

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