



## Nebraska Public Power District

COOPER NUCLEAR STATION  
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321  
TELEPHONE (402)825-3811  
FAX (402)825-5205

NLS960081

April 19, 1996

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555-0001

Dear Sir:

Cooper Nuclear Station Licensee Event Report 96-003 is forwarded as an attachment to this letter. A supplemental report with an expected submission date of May 20, 1996, will be submitted following additional investigation.

Sincerely,

  
J. T. Herron  
Plant Manager

/cct

Attachment

cc: Regional Administrator  
USNRC - Region IV

Senior Project Manager  
USNRC - NRR Project Directorate IV-1

Senior Resident Inspector  
USNRC

NPG Distribution

INPO Records Center

W. Turnbull  
MidAmerica Energy

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EXPIRES 04/30/98

## LICENSEE EVENT REPORT (LER)

(See reverse for required number of  
digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION  
COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED  
INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS  
REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT  
BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-  
0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF  
MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Cooper Nuclear Station	DOCKET NUMBER (2) 05000298	PAGE (3) 1 OF 1
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TITLE (4) Single Train RCIC System Inoperable Due to Equipment Malfunction
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EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
03	20	96	96	-- 003	-- 00	04	19	96	FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)	100	20.2201(b)	20.2203(a)(2)(v)	50.73(a)(2)(i)	50.73(a)(2)(viii)				
		20.2203(a)(1)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)				
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71				
		20.2203(a)(2)(ii)	20.2203(a)(4)	50.73(a)(2)(iv)	OTHER				
		20.2203(a)(2)(iii)	50.36(c)(1)	X 50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A				
20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)							

LICENSEE CONTACT FOR THIS LER (12)	
NAME Calvin C. Taylor, Licensing & Compliance Specialist	TELEPHONE NUMBER (Include Area Code) (402) 825-3811

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE).	NO		05	20	96

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At 1213 CST on March 20, 1996, the single train Reactor Core Isolation Cooling (RCIC) System was declared inoperable due to a failed monthly operability surveillance test. Upon RCIC pump turbine start, speed initially increased to approximately 4500 rpm (normal speed) but subsequently decreased and stabilized at approximately 2900 rpm. The governor valve was initially in the standby position of open and went towards the closed position as expected but did not ramp open. The governor valve output controller demand was one hundred percent but the governor valve indicated closed with the trip/throttle valve fully open. Control Room indication of flow was approximately 200 gpm. Normal flow is greater than 400 gpm.

Control Room operators initiated a RCIC turbine trip after receiving a low lube oil pressure alarm in accordance with the Alarm Response Procedure but control oil pressure was not sufficient to unlatch the trip/throttle linkage and the RCIC turbine was stopped by closing the RCIC steam supply valve.

A seven day LCO was entered due to the inoperable RCIC system. Trouble shooting determined an erroneous output from the Woodward EGM control box in the turbine governor control system. The EGM control box was replaced and satisfactorily tested and the RCIC system passed the monthly operability surveillance test. The seven day LCO was exited at 0508 on March 21, 1996.

Investigation of the failure is continuing and will be reported in a supplement to this LER.

LIST OF NRC COMMITMENTS	ATTACHMENT 3
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Correspondence No: NLS960081

The following table identifies those actions committed to by the District in this document. Any other actions discussed in the submittal represent intended or planned actions by the District. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Licensing Manager at Cooper Nuclear Station of any questions regarding this document or any associated regulatory commitments.

[illegible]