

NRC Form 366  
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Grand Gulf Nuclear Station - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 4 1 6

PAGE (3)

1 OF 03

TITLE (4)

Technical Specification Surveillance Not Performed

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)				
1	1	8	5	8	5	0	0	3	0	0	0	2		
1	1	8	5	8	5	0	0	3	0	0	0	2		
OK, RATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											
1			20.402(b)			20.406(c)			50.73(a)(2)(iv)			73.71(b)		
POWER LEVEL (10)			20.406(a)(1)(i)			50.36(a)(1)			50.73(a)(2)(v)			73.71(c)		
0 1 3 1 5			20.406(a)(1)(ii)			50.36(a)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
			20.406(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)					
			20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)					
			20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Ronald W. Byrd/License Engineer	6 0 1 4 3 7 1 - 2 1 4 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On five occasions from October 1984 to January 1985, Chemistry personnel failed to perform dose equivalent iodine/isotopic analyses following reactor scrams as required by Technical Specifications.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Grand Gulf Nuclear Station - Unit 1	0 5 0 0 0 4 1 6 8 5	-	0 0 3	-	0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Description of Reportable Occurrence

On five occasions from October 1984 to January 1985, Chemistry personnel failed to perform dose equivalent iodine/isotopic analyses following reactor scrams as required by Technical Specification Table 4.11.2.1.2-1.

Initial Conditions

At the time of discovery, January 18, 1985 at 1626 hours, the plant was at approximately 35 percent power operation.

Status of Redundant or Backup Systems

System ventilation monitors were operable and indicated some spikes occurring with the reactor scrams.

Nature of Occurrence

Technical Specification Table 4.11.2.1.2-1 requires certain radioactive gaseous release analyses to be performed following startup from cold shutdown, or following a thermal power change exceeding 15 percent of the Rated Thermal Power within a one hour period. This requirement is exempted if 1) analysis shows that the Dose Equivalent I-131 concentration in the primary coolant has not increased more than a factor of 3, and 2) the noble gas monitor shows that effluent activity has not increased more than a factor of 3. These analyses were not performed following reactor scrams from greater than 15% power from October 1984 to January 1985.

Immediate Corrective Actions Taken

Chemistry personnel were verbally instructed on the correct actions necessary to comply with the requirement. A formal log entry to clarify the requirements was promulgated to all chemists. The requirement to contact the Control Room every 4 hours was initiated and documented in the Chemistry Log. The On Shift chemist is required to verify power changes to ensure all required surveillances are completed within 2 to 6 hours as required by Technical Specifications.

Apparent Cause

The error made by utility Chemistry personnel was attributed in part to confusion on the application of the exemption statement of Technical Specification 3.4.5.C.1 which allows relaxation of the 15% power change requirement during the Startup Test Program. Neither the exemption nor plant procedures specify what values to use in determining the factor of 3. An additional contributing cause was poor communications between Control Room and Chemistry Lab personnel when power level changes were made.

NRC Form 365A  
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Grand Gulf Nuclear Station - Unit 1	0500041685	-003	-003	003	OF	03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Supplemental Corrective Action

Plant procedures are being written to specify the "normal" concentration and activity levels for various plant conditions. These values will be used for the comparison which determines if the analyses may be exempted.

In addition, a memorandum was issued to Plant Chemists describing the proper execution of Technical Specification requirements in accordance with the schedule or situation outlined in plant procedures.

Safety Assessment

The Technical Specification dose limits were not exceeded. All power level changes in excess of 15% in one hour were a result of reactor scrams. None of the missed analyses involved an increase in reactor power. Consequently, there was no case of potential increased releases without dose equivalent iodine/isotopic sampling being conducted.



# MISSISSIPPI POWER & LIGHT COMPANY

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P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

February 18, 1985

NUCLEAR LICENSING & SAFETY DEPARTMENT

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station  
Unit 1  
Docket No. 50-416  
License No. NPF-29  
File: 0260/L-835.0  
Technical Specification  
Surveillance Not Performed  
LER 85-003-0  
AECM-85/0053

Attached is Licensee Event Report (LER) 85-003-0 which is a final report.

Yours truly,

*Lam H. Hobbz*

L. F. Dale  
Director

*for*

EBS/SHH:vog  
Attachment

cc: Mr. J. B. Richard (w/a)  
Mr. R. B. McGehee (w/a)  
Mr. N. S. Reynolds (w/a)  
Mr. G. B. Taylor (w/o)

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