

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) BYRON UNIT 1										DOCKET NUMBER (2) 05000454										PAGE 1 OF 012																													
TITLE (4) MAIN STEAM SAFETY VALVES OUT OF TOLERANCE																																																	
EVENT DATE (5) MONTH DAY YEAR 013185										LER NUMBER (6) YEAR SEQUENTIAL NUMBER REVISION NUMBER 85-015-0002										REPORT DATE (7) MONTH DAY YEAR 1985										OTHER FACILITIES INVOLVED (8) FACILITY NAMES DOCKET NUMBER(S) 05000000																			
OPERATING MODE (9) 3										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. Check one or more of the following: (11)																																							
POWER LEVEL (10) 01010										20.402(b) 20.406(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iv) 20.406(a)(1)(v)										20.406(a) 90.38(a)(1) 90.38(a)(2) 90.73(a)(2)(i) 90.73(a)(2)(ii) 90.73(a)(2)(iii)										90.73(a)(2)(iv) 90.73(a)(2)(v) 90.73(a)(2)(vi) 90.73(a)(2)(vii)(A) 90.73(a)(2)(vii)(B) 90.73(a)(2)(vii)(C)										73.71(b) 73.71(a) OTHER (Specify in Abstract below and in Test, NRC Form 308A)									
LICENSEE CONTACT FOR THIS LER (12) NAME Leo Wehner, System Test Engineer (extension 384)																														TELEPHONE NUMBER AREA CODE 8152345441																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRC										CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRC																																							
X S B R V C 5 6 7 Y																																																	
SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE:) X NO																														EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR																			

ABSTRACT (Limit to 1400 spaces or approximately fifteen single-space typewritten lines) (16)

During a surveillance on the Main Steam Safety valves, five valves were found to be outside of the acceptable lift setting range. The valves were immediately adjusted to the proper setting and returned to operable status.

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PDR ADOCK 05000454
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 9/31/85

FACILITY NAME (1) BYRON UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 4 5 4	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	— 0 1 5	— 0 0	0 1	OF 0 2

TEXT (If more space is required, use additional NRC Form 386A's) (17)

During the performance of the Main Steam Safety Valve surveillance on January 30 and 31 five safety valves were found to have a lift setting outside the $\pm 1\%$ acceptable tolerance. The plant was operating in Mode 3 at the time that the safety valves were being tested. The surveillance is performed every 18 months on each main steam safety valve.

The out of tolerance lift setting can be attributed to the construction activity which has been on going adjacent to the safety valves over the past 18 months. The safety valves were initially set on June 11, 1983.

There were no safety consequences or implications as a result of the out of tolerance lift settings. No more than two safety valves on any steam generator were in the out-of-tolerance. The safety valve lift settings were immediately reset within the required range.

The valves involved are Consolidated 3707R Safety Valves. This was the initial performance of the surveillance.



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

DATE: February 19, 1985

LTR: BYRON 85-0262

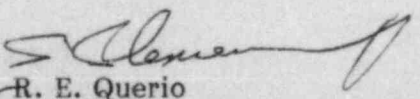
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR 50.73(a)(2)(iv) which requires a 30 day written report.

This report is number 85-015-00, Docket No. 50-454.

Very truly yours,


R. E. Querio
Station Superintendent
Byron Nuclear Power Station

Enclosure: Licensee Event Report No. 85-017-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
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