

Commonwealth Edison Company
LaSalle Generating Station
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Marseilles, IL 61341-9757
Tel 815-357-6761



April 11, 1996

**United States Nuclear Regulatory Commission
Washington, D.C. 20555**

Attention: Document Control Desk

Subject: LaSalle County Station Units 1 and 2
Monthly Performance Report
NRC Docket Numbers 50-373 and 50-374.

Enclosed is the LaSalle County Station Monthly Performance Report for the month of March, 1996.

Respectfully,

A handwritten signature in dark ink, appearing to read "D. J. Ray".

D. J. Ray
Station Manager
LaSalle County Station

Enclosure

cc: H. J. Miller, NRC Region III Administrator
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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

March 1996

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION

II. REPORT

A. SUMMARY OF OPERATING EXPERIENCE

B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815) 357-6761, extension 2056.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor sub-critical, Generator off-line, refuel outage (L1R07) in progress.
31	2400	Reactor sub-critical, Generator off-line, refuel outage (L1R07) in progress.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On March 11, 1996, Amendment 110 was issued to license NPF-11 (Unit 1). This amendment allows the implementation of the new 10 CFR 50 Appendix J leak testing requirements.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 1)

<u>LER No.</u>	<u>Occurrence Date</u>	<u>Description</u>
96-001	02/13/96	Four fire protection system valves were not verified to be in the correct position as required by the Technical Specification.

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(See Attachment A)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE April 10, 1996
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: March 1996
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

GROSS HOURS IN REPORTING PERIOD 744
MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-N 1,078

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	585.5	75,527.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	0.0	582.3	73,922.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWht)	0	1,411,768	219,866,585
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	0	470,404	73,514,256
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	-8,246	432,896	70,571,324
12. REACTOR SERVICE FACTOR (%)	0.0	26.8	70.3
13. REACTOR AVAILABILITY FACTOR (%)	0.0	26.8	71.9
14. UNIT SERVICE FACTOR (%)	0.0	26.7	68.8
15. UNIT AVAILABILITY FACTOR (%)	0.0	26.7	68.8
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	19.1	63.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	-1.0	18.4	61.0
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	7.8

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refuel, 01/25/96, 10 Weeks
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 04/09/96

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE April 10, 1996
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: March 1996

DAY	POWER	DAY	POWER
1	-11	17	-11
2	-11	18	-11
3	-11	19	-11
4	-11	20	-11
5	-11	21	-11
6	-11	22	-11
7	-11	23	-11
8	-11	24	-11
9	-11	25	-11
10	-11	26	-11
11	-11	27	-11
12	-11	28	-11
13	-11	29	-11
14	-11	30	-12
15	-11	31	-12
16	-11		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
01	960125	S	744.0	C	2	Refuel outage (L1R07)

SUMMARY OF OPERATION: The unit remained in a scheduled refueling outage for the entire month.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

March 1996

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS
(UNIT 2)

- I. INTRODUCTION
- II. REPORT
 - A. SUMMARY OF OPERATING EXPERIENCE
 - B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS
 - C. LICENSEE EVENT REPORTS
 - D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
 - E. UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Safety Relief Valve Operations
 - 2. Major Changes to Radioactive Waste Treatment System
 - 3. Static O-Ring Failures
 - 4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815) 357-6761, extension 2056.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1135 Mwe.
9	2320	Reduced power level to 1020 Mwe to transfer the Circulating Water system pumps and the Condensate/Condensate Booster pumps.
10	0130	Reduced power level to 850 Mwe for performance of a rod set.
	1600	Increased power level to 1130 Mwe.
30	2100	Reduced power level to 750 Mwe for performance of a rod set.
31	1100	Increased power level to 1030 Mwe, power level held due to core limits.
	2400	Reactor critical, Generator on-line at 1020 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On March 11, 1996, Amendment 95 was issued to license NPF-18 (Unit 2). This amendment allows the implementation of the new 10 CFR 50 Appendix J leak testing requirements.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 2)

<u>LER No.</u>	<u>Occurrence Date</u>	<u>Description</u>
96-002	02/04/96	Manual Reactor scram due to loss of cooling on the '2E' Main Power Transformer.

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE April 10, 1996
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: March 1996 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY): N/A

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	2,089.3	73,378.9
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	2,020.2	71,861.7
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWht)	2,442,127	6,372,259	218,341,514
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	839,556	2,173,159	73,071,015
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	816,216	2,103,505	70,256,177
12. REACTOR SERVICE FACTOR (%)	100.0	95.7	73.1
13. REACTOR AVAILABILITY FACTOR (%)	100.0	95.7	74.8
14. UNIT SERVICE FACTOR (%)	100.0	92.5	71.6
15. UNIT AVAILABILITY FACTOR (%)	100.0	92.5	71.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	105.9	93.0	67.6
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	101.8	89.3	64.9
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	7.5	10.0

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refuel, 09/07/96, 10 Weeks

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE April 10, 1996
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: March 1996

DAY	POWER	DAY	POWER
1	1,105	17	1,107
2	1,103	18	1,106
3	1,103	19	1,105
4	1,103	20	1,104
5	1,101	21	1,105
6	1,101	22	1,107
7	1,100	23	1,110
8	1,099	24	1,110
9	1,096	25	1,105
10	1,045	26	1,105
11	1,107	27	1,106
12	1,108	28	1,106
13	1,108	29	1,105
14	1,107	30	1,079
15	1,106	31	949
16	1,106		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
(None)						

SUMMARY OF OPERATION: The unit remained on-line at high power throughout the month. Several minor power reductions were required throughout the month for routine maintenance activities.

Attachment A

Safety Relief Valve Operations

The following operations took place with the unit shutdown and with the reactor depressurized as part of routine intentional operability testing.

<u>Valve</u>	<u>Date of Actuation</u>	<u>Type of Actuation</u>	<u>Reason for Actuation</u>
1B21F013A	03/19/96	Manual	Operability Testing
1B21F013B	03/19/96	Manual	Operability Testing
1B21F013C	03/19/96	Manual	Operability Testing
1B21F013D	03/19/96	Manual	Operability Testing
1B21F013E	03/19/96	Manual	Operability Testing
1B21F013F	03/19/96	Manual	Operability Testing
1B21F013G	03/19/96	Manual	Operability Testing
1B21F013H	03/19/96	Manual	Operability Testing
1B21F013J	03/19/96	Manual	Operability Testing
1B21F013K	03/19/96	Manual	Operability Testing
1B21F013L	03/19/96	Manual	Operability Testing
1B21F013M	03/19/96	Manual	Operability Testing
1B21F013N	03/19/96	Manual	Operability Testing
1B21F013P	03/19/96	Manual	Operability Testing
1B21F013R	03/19/96	Manual	Operability Testing
1B21F013S	03/19/96	Manual	Operability Testing
1B21F013U	03/19/96	Manual	Operability Testing
1B21F013V	03/19/96	Manual	Operability Testing
1B21F013A	03/28/96	Manual	Operability Testing
1B21F013B	03/28/96	Manual	Operability Testing
1B21F013C	03/28/96	Manual	Operability Testing
1B21F013D	03/28/96	Manual	Operability Testing
1B21F013E	03/28/96	Manual	Operability Testing
1B21F013F	03/28/96	Manual	Operability Testing
1B21F013H	03/28/96	Manual	Operability Testing
1B21F013J	03/28/96	Manual	Operability Testing
1B21F013K	03/28/96	Manual	Operability Testing
1B21F013L	03/28/96	Manual	Operability Testing
1B21F013N	03/28/96	Manual	Operability Testing
1B21F013R	03/28/96	Manual	Operability Testing
1B21F013S	03/28/96	Manual	Operability Testing
1B21F013V	03/28/96	Manual	Operability Testing
1B21F013U	03/29/96	Manual	Operability Testing