

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 1 6				PAGE (3) 1 OF 0 2		
TITLE (4) ESF ACTUATION DUE TO HIGH DIFFERENTIAL TEMPERATURE IN RWCU ROOM																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 9	0 8	8 4	8 4	0 0 7	0 0	1 0	0 5	8 4					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)														
POWER LEVEL (10)		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)		
0 6 1 9		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)		
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME T. L. Elton, Acting Supt. of Regulatory Compliance										TELEPHONE NUMBER 9 1 1 2 3 1 6 1 7 + 1 7 8 5 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 09/08/84, at approximately 2317 CDT, with the reactor mode switch in the run position and reactor power at 1691 MWT (approximately 69% power), the Reactor Water Clean-up (RWCU) Inboard Isolation Valve (2G31-F001) isolated due to high differential temperature in the RWCU pump room. No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 2 systems or on Unit 1. The health and safety of the public were not affected by this non-repetitive event.

The cause of this event is an inboard seal leak on the 2A RWCU pump.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT 2	0500036684	84	007	000	02	OF	02

TEXT (if more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR 50.73 (a)(2)(IV) since the Reactor Water Clean-up Inboard Isolation Valve (2G31-F001) is a Primary Containment Isolation Valve and its isolation constitutes the actuation of an Engineered Safety Feature (ESF).

On 09/08/84, at approximately 2317 CDT, with the reactor mode switch in the run position and reactor power at 1691 MWt (approximately 69% power), the Reactor Water Clean-up (RWCU) Inboard Isolation Valve (2G31-F001) isolated due to high differential temperature in the RWCU pump room. No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 2 systems or on Unit 1. The health and safety of the public were not affected by this non-repetitive event.

The cause of this event was an inboard seal leak on the 2A RWCU pump. The RWCU inboard isolation valve (2G31-F001) was re-opened on 09/10/84. The 2A RWCCU pump was repaired and returned to service on 09/10/84.

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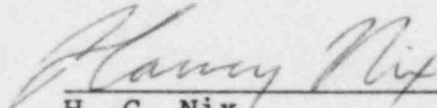
Edwin I. Hatch Nuclear Plant

October 5, 1984
GM-84-863

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-366/1984-07. This report is required by 10 CFR 50.73(a)(2)(IV).


H. C. Nix
General Manager

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HCN/TLE/vlt

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