

NRC Form 365
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Pilgrim Nuclear Power Station (PNPS) - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 2 9 3 1 OF 0 2

PAGE (3)

TITLE (4)

Safety Valve Setpoints Below Requirement of Technical Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)	
03	28	84	84	004	01	10	10	84			0 5 0 0 0	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
N												
POWER LEVEL (10)			20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)
0 1 0 0			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)			
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)			
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)			

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
R. Schifone - Plant Engineer	AREA CODE 617 714 61-79 00

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD
D	S	B	R	V	D	2	4	3	Y

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 3/28/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the Main Steam Safety Valves exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi \pm 13 psi. When tested, one valve lifted at 1209 psi, and the other lifted at 1155 psi.

The cause of this deviation has been determined to be the set pressure calibration method. The procedure allowed for the use of the nitrogen as a substitute test gas, in lieu of steam.

As a result of this determination, future safety valve testing/calibration will be performed using steam as the test medium.

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NRC Form 366A
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Pilgrim Nuclear Power Station - Unit 1	0500029384	—	004	—	01	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 3/28/84, during a refueling outage, the Maintenance Department was notified by Wyle Laboratories that both of the PNPS Dresser Main Steam Safety Valves, Model 3777, exhibited set pressures more than 1% below the nameplate set pressure. This is contrary to the requirements of PNPS Technical Specification (T.S.) 2.2.C, which requires both valves to lift at 1240 psi \pm 13 psi. The subject valves were being tested in accordance with the requirements of T.S. 4.6.D.1.

Both valves were tested twice. Valve #203-4A, Serial #BK6262 lifted at 1213 psi during Test 1 and 1209 psi during Test 2.

Valve #203-4B, Serial #BK6309 lifted at 1165 psi during Test 1 and 1155 psi during Test 2.

Subsequent evaluation has indicated that the procedure used for set pressure calibration was inadequate. The procedure allowed for the use of nitrogen as a test gas, in lieu of steam. It has been concluded that this substitution resulted in the safety valves' pressure setpoints being in error.

On 4/7/84, at Wyle Lab., the above-mentioned valves were calibrated for set pressure and tested for leakage with steam as the test medium. Both valves were certified as acceptable.

As a result of these findings, future safety valve testing/calibration will be performed with steam as the test medium as indicated by Station Procedure 3.M.4-7.

This event did not impact the health and safety of the public.

A search of records indicates no previous occurrences of a similar nature.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

October 10, 1984

BECO Ltr. #84-171

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket Number 50-293
License DPR-35

Dear Sir:

The attached update Licensee Event Report 84-004-01, "Safety Valve Setpoints Below Requirement of Technical Specifications," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

W.D. Harrington

W. D. Harrington

PH:caw

Enclosure: LER 84-004-01

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECO LER Distribution

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