

Docket No. 50-346

License No. NPF-3

Serial No. 1-469

October 5, 1984



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Mr. C. E. Norelius, Director
Division of Reactor Projects
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Norelius:

Toledo Edison acknowledges receipt of your September 11, 1984 letter (Log No. 1-1031), and enclosures, Appendix A, Notice of Violation, and Inspection Report No. 50-346/E4-18 (DRP).

Following an examination of the items of concern, Toledo Edison herein offers information regarding these items:

1. Violation: 10 CFR 50 Appendix B, Criterion III, states in part:
"Measures shall be established to assure that applicable regulatory requirements and the design basis, as defined in 10 CFR 50.2 and as specified in the license application, for those structures, systems and components to which this appendix applies are correctly translated into specifications, drawings, procedures, and instructions... Design changes, including field changes, shall be subject to design control measures commensurate with those applied to the original design..."

Contrary to the above, the design basis power supply for reactor coolant system high point vent valve RC 4608A was not correctly translated into the drawings and instructions included in Facility Change Request (FCR) 80-120 for supplying power to valve RC 4608A. The design basis power supply for RC 4608A was Class 1E power. The drawings and instructions included in FCR 80-120 erroneously required RC 4608A to receive non-Class 1E power.

This is a Severity Level IV violation (Supplement I).

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Response:(1) Corrective action taken and results achieved.

A review of the appropriate drawings was conducted to confirm that a wiring problem existed. The Station Technical Section was notified by Surveillance Report 84-30 to determine reportability.

Drawing Change Notices were issued by Bechtel and a supplement to the Facility Change Request (FCR) 80-120 was initiated to provide an essential source of DC power to SV 4608A.

Bechtel has reviewed the power supplies to the other three solenoid valves installed under FCR 80-120 as well as power supplies to all Class 1E relay cabinets. No discrepancies were discovered.

(2) Corrective action to be taken to avoid further noncompliance.

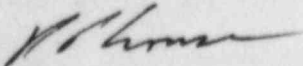
Based on Bechtel's review, the design error was an isolated case and there was no breakdown in their design verification procedures.

Additionally, to ensure that future modifications are reviewed for proper power supplies, the Loss of Instrument Power (LIP) review sheet has been revised to show a separate check-off block for a review to verify safety-related components have an essential source of power. Procedure NFEP-020, Design Work Packages, was revised to require the incorporation of the revised LIP into future applicable work packages.

(3) Date when full compliance will be achieved.

Full compliance will be achieved upon implementation of the supplement to FCR 80-120, which will occur by the end of the 1984 refueling outage.

Very truly yours,



RPC:TPB:SGW:nlf
cc: DB-1 NRC Resident Inspector