

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Quad-Cities Nuclear Power Station										DOCKET NUMBER (2) 0 5 0 0 0 2 5 4 1				PAGE (3) 1 OF 01		
TITLE (4) Standby Gas Treatment System Auto-Start																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES NA				DOCKET NUMBER(S) 0 5 0 0 0			
09	24	84	84	0118	00	10	11	84					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
POWER LEVEL (10)		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 60.73(a)(2)(iv)				73.71(b)		
		20.405(a)(1)(i)				60.38(c)(1)				<input type="checkbox"/> 60.73(a)(2)(v)				73.71(e)		
		20.405(a)(1)(iii)				60.38(c)(2)				<input type="checkbox"/> 60.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)		
		20.405(a)(1)(iii)				60.73(a)(2)(i)				<input type="checkbox"/> 60.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				60.73(a)(2)(ii)				<input type="checkbox"/> 60.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				60.73(a)(2)(iii)				<input type="checkbox"/> 60.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Gary F. Spedl, Technical Staff Supervisor										TELEPHONE NUMBER AREA CODE 310 9 6151 41-1212 41						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		1	02	28	4	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 24, 1984, at 10:30 a.m., both Unit 1 and Unit 2 were at approximately 99% core thermal power. The Reactor Building Fuel Pool Channel B Area Radiation Monitor (IL), 1705-16B, spiked high tripping the Reactor Building Ventilation (VA) and auto-starting the Standby Gas Treatment System (BN). The B Channel monitor had returned to normal when checked and no abnormal radiation levels were found on the Refuel Floor during the survey conducted by Radiation Protection Department. The Reactor Building Ventilation System was returned to normal and Standby Gas Treatment System was secured.

This revision 00 of Licensee Event Report 84-18 was initiated to document the undesirable event described for the Station's records. Since an identical event had happened on September 22, 1984, no separate evaluation was deemed necessary for the September 24 occurrence. All evaluations and necessary corrective actions for both September 22 and September 24 events will be documented in a supplemental report to L.E.R. 84-18.

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