NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

# LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | DOCKET NUMBER (2) | PAGE (3) |
|---|---|---|
| BYRON STATION | 0 5 0 0 0 4 5 4 | 1 OF 0 2 |

TITLE (4)

1A STEAM GENERATOR LOW LOW LEVEL

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| 0 1 | 0 4 | 8 5 | 8 5 | — 0 0 8 | — 0 0 | 0 2 | 0 1 | 8 5 | | 0 5 0 0 0 |
| | | | | | | | | | | 0 5 0 0 0 |

| OPERATING MODE (9) | 4 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | |
|---|---|---|---|---|---|
| POWER LEVEL (10) | 0 1 0 0 | 20.402(b) | 20.405(c) | X 50.73(a)(2)(iv) | 73.71(b) |
| | | 20.405(a)(1)(i) | 50.36(c)(1) | 50.73(a)(2)(v) | 73.71(c) |
| | | 20.405(a)(1)(ii) | 50.36(c)(2) | 50.73(a)(2)(vii) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| | | 20.405(a)(1)(iii) | 50.73(a)(2)(i) | 50.73(a)(2)(viii)(A) | |
| | | 20.405(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | |
| | | 20.405(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

| NAME | TELEPHONE NUMBER |
|---|---|
| Larry Alexander, Tech Staff Engineer, Ext. 388 | AREA CODE 8 1 5 2 3 4 - 5 4 4 1 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | |
|---|---|---|---|---|---|---|---|---|---|---|---|
| A | J B | I S G | / / / / | N | | | | | | | |
| | | | | | | | | | | | |

| SUPPLEMENTAL REPORT EXPECTED (14) | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|---|---|---|---|---|
| YES (If yes, complete EXPECTED SUBMISSION DATE) | X NO | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

With the plant in mode 4, a reactor trip and Auxiliary Feedwater actuation occurred due to a Steam Generator low-low level alarm. The operators inadvertently allowed the level to drop in the Steam Generator while engaged in a shift turnover and in a Main Steam valve lineup to drain condensation. The Operating Staff is being appraised for this event through the required reading program.

IE22
1/1



NRC Form 366
(9-83)

| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | PAGE (3) |
|---|---|---|---|---|---|
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | |
| BYRON, UNIT 1 | 0 5 0 0 0 4 5 4 | 8 5 | — 0 0 8 | — 0 0 | 0 2 OF 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 1/4/85 at 2248, with the plant operating in Mode 4, the 1A Steam Generator experienced a low-low level alarm resulting in a reactor trip and Aux Feedwater actuation. Prior to the event, the Steam Generator Blowdown System was valved in at 1800 to decrease the level in the Steam Generators in preparation for heat-up. At 1849, Steam Generator Blowdown was secured and at 2115 the 1A Steam Generator low level alarm annunciated in the Main Control Room. The Unit Operator suspected a leaky blowdown valve and manually isolated the system at this time. However, the Unit Operator, involved in his shift turnover, did not recheck his level instrumentation (assuming the blowdown isolation corrected the problem), and at 2248 the low-low Steam Generator level alarm initiated the reactor trip. In the time between the low level and the low-low level alarms, a Main Steam valve line-up to drain condensation that might have formed in the steam lines was being conducted. This valve line-up took long enough to create the additional level loss in the 1A Steam Generator due to condensation. There was no indication of an increase in water inventory in the Blowdown System.

Plant and public safety were not affected by this event since the control rods were fully inserted and there was no decay heat in the core. The operating staff is being appraised of this event through the station's required reading program.

Previous occurrences: None

**Commonwealth Edison**
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

February 1, 1985

BYRON LTR: 85-0170

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR 50.73 (a)(2)(iv) which requires a 30 day written report.

This report is number 85-008-00, Docket No. 50-454.

Very truly yours,

R. E. Querio
Station Superintendent
Byron Nuclear Power Station

Enclosure: Licensee Event Report No. 85-008-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
CECo Distribution List

IE22
1/1