

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Joseph M. Farley - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 6 4										PAGE 13 1 OF 0 2	
TITLE (4) Action Statements Not Met For Inoperable Steam Flow Channel																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)						
0 9	1 5	8 4	8 4	0 1 0	0 0	1 0	1 5	8 4							0 5 0 0 0						
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																			
1		20.402(b)				20.406(a)				20.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)		20.406(a)(1)(i)				20.30(a)(1)				20.73(a)(2)(v)				73.71(a)							
0 3 2		20.406(a)(1)(ii)				20.30(a)(2)				20.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 305A)							
		20.406(a)(1)(iii)				20.73(a)(2)(i)				20.73(a)(2)(vii)(A)											
		20.406(a)(1)(iv)				20.73(a)(2)(ii)				20.73(a)(2)(vii)(B)											
		20.406(a)(1)(v)				20.73(a)(2)(iii)				20.73(a)(2)(viii)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME J. D. Woodard										TELEPHONE NUMBER											
										AREA CODE 2 0 5 8 9 9 - 5 1 5 6											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If you complete EXPECTED SUBMISSION DATE)												X NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1731 on 9-15-84, during a power increase after a maintenance outage, steam generator 2A steam flow channel FI-475 was found to be inoperable because it exceeded the allowable tolerance for difference between redundant channels. Technical Specification action statements require an inoperable channel to be placed in the tripped condition within one hour. However, the channel was not placed in the tripped condition until 2125 on 9-15-84. The equalizing valve on the flow transmitter for FI-475 was found to be not fully closed. This valve was closed and the channel was declared operable at 2309 on 9-15-84. Health/safety of the public were not affected.

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PDR ADDOCK 05000364
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Joseph M. Farley - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 4 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	010	00	02	OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

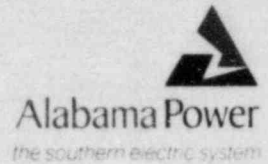
On 9-15-84, the power level on Unit 2 was being increased after a maintenance outage. At 1731, with the unit operating at 32% power, steam generator 2A steam flow channel FI-475 was found to be inoperable when the indicated flow on this channel was lower than that of other channels and the allowable tolerance between redundant channels was exceeded. Technical Specification action statements require the inoperable channel to be placed in the tripped condition within one hour. However, the Shift Supervisor did not recognize that a Limiting Condition for Operation was not met and the channel was not placed in the tripped condition until 2125 on 9-15-84.

The equalizing valve on the flow transmitter for FI-475 was found to be not fully closed but the condition of the valve made it appear closed when it was manually checked. The packing nut on the equalizing valve had backed off. When the valve handle was turned to close the valve, the handle would tighten against the packing nut and the valve would appear to be closed when it was only partially closed. The valve was repaired and closed allowing FI-475 to give proper indication. The channel was declared operable at 2309 on 9-15-84.

This event was caused by personnel error. The Shift Supervisor did not pay proper attention to detail and did not recognize that a Limiting Condition for Operation was not met when FI-475 was inoperable. The Shift Supervisor involved in this event has been counseled.

Mailing Address
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Post Office Box 2641
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R. P. McDonald
Senior Vice President
Flintridge Building



October 15, 1984

Docket No. 364

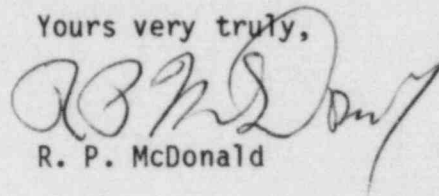
Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 2, Licensee Event Report No. LER 84-010-00 is forwarded in accordance with 10CFR50.73 to provide 30 day written notification of the occurrence.

If you have any questions, please advise.

Yours very truly,



R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

IE22
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