

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) BYRON, UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 4 5 4										PAGE (3) 1 OF 02			
TITLE (4) DEMAND POSITION INDICATION FAILURE																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)								
1	2	2	5	8	4	8	4	0	4	0	0	0	1	2	4	8	5	0 5 0 0 0					
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																							
OPERATING (10)		5		20.402(b)				20.402(c)				X 80.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10)		0 0 0		20.402(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(iv)				73.71(a)							
				20.402(a)(1)(ii)				80.36(a)(2)				80.73(a)(2)(v)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)							
				20.402(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(iv)(A)											
				20.402(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(iv)(B)											
				20.402(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(ii)											
LICENSEE CONTACT FOR THIS LER (12)																							
NAME DON GEORGE (x-608)										TELEPHONE NUMBER													
										AREA CODE 8 1 5 2 3 4 - 5 4 4 1													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC													
C	C D	X I	W 1 2 0	N																			
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)													
YES (If you complete EXPECTED SUBMISSION DATE)										X NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The Digital Rod Position Indication and the Demand Step Counters differed by more than 12 steps, and therefore the reactor trip breakers were manually opened. The cause of the event was a step counter sticking. The faulty step counter was replaced and all step counters were cleaned and lubricated.

8502010843 850124
PDR ADOCK 05000454
S PDR

IE22
4/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) BYRON, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 4 5 4 8 4	LER NUMBER (8)			PAGE (7)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		—	0 4 0	— 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 308A's) (17)

On December 15, 1984 at approximately 02:13, during the execution of the Digital Rod Position Indication (DRPI) System Surveillance, DRPI and Demand indication of control rod position differed by more than 12 steps for Control Bank A group 1 rods. The plant was in Mode 4. Per the ACTION statement in Technical Specification Special Test Exception 3/4.10.5, the Reactor Trip Breakers were immediately opened.

While Control Bank A was being inserted per the DRPI Operability Surveillance it was noticed that the DRPI and Demand indication (step counter) were not agreeing. The NSO stopped rod motion when DRPI was eleven steps out of alignment with the step counters. The NSO then moved rods in from 203 to 199 steps per the counter, and DRPI indication changed from 192 to 185 such that the DRPI versus the counter indication were 13 steps out of alignment. The reactor trip breakers were immediately opened at this time.

DRPI versus demand checks have shown, in practice, that the greatest difference is nominally four steps. The mismatch in this event was caused by the step counter sticking during either withdrawal or insertion.

There was no safety impact since the step counter malfunction did not affect the actual rod position.

The maintenance department has cleaned and lubricated all step counters and also replaced the Control Bank A group 1 step counter.

There have been no previous occurrences of this type (DRPI versus Demand more than 12 steps out).



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

January 24, 1985

LTR: BYRON 85-0122

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2)(iv) which requires a 30 day written report.

This report is number 84-040-00, Docket No. 50-454.

Very truly yours,

R. E. Querio
Station Superintendent
Byron Nuclear Power Station

REQ/vda

Enclosure: Licensee Event Report No. 84-040-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
CECO Distribution List

IEP2
1/1