

TENNESSEE VALLEY AUTHORITY  
Division of Power Production

SECONDARY CONTAINMENT  
LEAK RATE TEST  
BROWNS FERRY NUCLEAR PLANT UNIT 2  
CONDUCTED MARCH 20, 1978

SUBMITTED TO  
THE UNITED STATES NUCLEAR REGULATORY COMMISSION  
PURSUANT TO  
FACILITY OPERATING LICENSE NUMBER DPR-52

7903190271

# 1.0 - Browns Ferry Nuclear Plant Unit 2 Secondary Containment Leak Rate Test Report

## 2.0 Purpose

This report describes the results and analysis of the test data taken during leak rate testing of the Browns Ferry Nuclear Plant Unit 2 secondary containment pursuant to Technical Specification 4.7.C.1.c prior to the first refueling outage.

## 3.0 Procedure

The attached surveillance instruction, SI 4.7.C-1, outlines the procedures followed during secondary containment leak rate testing.

## 4.0 Data

The attached surveillance instruction data sheets lists the following test data:

- (1) Standby gas treatment system flow rate: 8700 scfm
- (2) Reactor building differential pressure:
  - Unit 2 reactor zone:  $-0.53'' \text{ H}_2\text{O}$
  - Refueling zone average:  $-0.48'' \text{ H}_2\text{O}$
- (3) Wind Speed: 17.1 mph
- (4) Wind direction: southwest

## 5.0 Analysis and Interpretation

Technical Specification 4.7.C.1.c requires that secondary containment capability to maintain 1/4 inch water vacuum under calm (<5 mph) wind conditions with a total system inleakage rate of not more than 12000 scfm shall be demonstrated at each refueling outage prior to refueling. Following shutdown of Unit 2 on March 18, 1978, the secondary containment for the common refuel zone and the Unit 2 reactor zone were leak rate tested.

Browns Ferry Nuclear Plant Unit Secondary Containment Leak Rate Test  
Report (Continued)

5.0 Analysis and Interpretation (Continued)

These two zones were isolated from the Unit 1 and Unit 3 reactor zones. Appendix A to SI 4.7.C gives the allowable inleakage per secondary containment zone documented during preoperational testing. Assuming a total allowable inleakage of 12000 scfm, it is shown in Appendix A that the allowable inleakage for testing the refuel zone and Unit 2 reactor zone in combination under calm wind conditions is 8988 scfm. From the data taken during this test it is shown that assuming a total combined inleakage of 8700 scfm in the Unit 2 reactor zone and the refuel zone that the secondary containment system is capable of maintaining greater than 0.25 inches of water vacuum. These results fulfill the requirements of Technical Specification 4.7.C.1.c.

SI 4.7.C - Secondary Containment

Description

This surveillance is used to comply with the requirements of portions of technical specification 4.7.C. The following table lists the requirements satisfied by this instruction

<u>Frequency</u>	<u>Tech. Spec. Reference</u>	<u>Surveillance Requirements</u>
At each refueling outage prior to refueling	4.7.C.1.c	Demonstrate the capability of secondary containment to maintain -1/4" water pressure with a system inleakage of not more than 12,000 cfm under calm wind conditions $\leq$ 5 mph. SI 4.7.C-1 will be used to satisfy this requirement.
Any time the integrity of one zone has been violated	4.7.C.2	Isolate the affected zone from the other zones and demonstrate secondary containment capability to maintain -1/4" water pressure under calm wind conditions using the SGTS. SI 4.7.C-1 will be used to satisfy the requirements for the zones.

The reactor building is broken up into four zones, each of which can be isolated independently. These zones are the unit 1 reactor zone (U1RZ), the unit 2 Reactor zone (U2RZ), the unit 3 reactor zone (U3RZ), and the refueling zone (RFZ) which is common to all three units.

This surveillance instruction is written to use the SGTS fans A and C, B and C, or A and B, or a single fan to demonstrate that at a flow of  $\leq$  12,000 cfm the secondary containment zones can be maintained at a static pressure of -1/4" water under calm wind conditions. Manual isolation of the zones will be used since it is fast and the steam line tunnel temperature increases, which may cause a unit trip.

SI 4.7.C - Secondary Containment

Description (Continued)

SI 4.7.C-1 will be used to demonstrate secondary containment capability. This is to be accomplished by isolating the secondary containment for the reactor building or the required zones, starting the standby gas treatment trains, adjusting the flow to less than the allowable flow and verifying the static pressure of the tested zone(s) is  $-1/4''$  H<sub>2</sub>O or less. This procedure will normally be used to test the secondary containment capability of the reactor building prior to refueling.

SI 4.7.C-1 Testing should be limited to 30 minutes if at all possible since the main steam line tunnel temperatures may trip an operating unit.

SI 4.7.C - Secondary Containment Capability

1. PURPOSE

This procedure is used to isolate the zones to be leak tested and to verify secondary containment capability to maintain  $-1/4"$  H<sub>2</sub>O with a system in-leakage of  $\leq 12,000$  cfm without actually measuring the in-leakage.

2. REFERENCES

2.1 Technical specifications for units 1, 2, and 3, Section 4.7.C.

2.2 The following drawings:

<u>Number</u>	<u>Revision</u>	<u>Number</u>	<u>Revision</u>
45N614-5	5	47W610-64	15
45N614-6	5	47W610-65	15

3. PREREQUISITES

3.1 Notify each unit operator and assistant shift engineer such that they may communicate and be aware of the ventilation and SGTS status during this test.

3.2 Verify that the reactor building ventilation system is in a normal operational status per OI 30.

3.3 Verify that the SGTS is in standby readiness per OI 65.

3.4 Verify that each unit's main steam line tunnel temperature is not above 150° F.

3.5 Verify switch 16A-S34 is not in DRYWELL BYPASS or TORUS BYPASS on each unit or the unit that is to be tested.

SI 4.7.C-1 - Secondary Containment Capability

3. PREREQUISITES (Continued)

3.6 Station a man at the Reactor Building 480V Vent Board in case any breakers need to be reset during this test, and in the SGTS building to adjust manual dampers.

3.7 Set up communication between panel 9-25-1 and 9-25-2 and the following

panels:

25-219-1
25-215-2
25-215-3
25-213-1
25-213-2
25-213-3

3.8 Verify the following process instruments to be within a current

calibration period.	FI-65-50	1-PdIC-64-2
	FI-65-71	2-PdIC-64-2
	1-PdIC-64-1	3-PdIC-64-2
	2-PdIC-64-1	
	3-PdIC-64-1	

3.9 Verify the inboard equipment access lock doors closed with seals inflated.

3.10 Verify that a drywell or suppression chamber purge is not in process or planned during this test.

3.11 Verify on Data Sheet SI 4.7.C-1 that each of the doors are closed for the respective zones to be tested or the doors indicated by number (See Table 3.11 of Data Sheet SI 4.7.C-1) for all four zones.

4. PRECAUTIONS

4.1 Do not allow the main steam line tunnel temperature on any unit in operation to exceed 160° F. If this temperature is approached to within 10° F, stop the test and reestablish normal ventilation per OI 30 or provide another means for ventilating the main steam line tunnel to prevent a unit trip (trip point is 186° F).

4.2 Attempt to restrict testing time to 30 minutes.

5. LIMITATIONS AND ACTIONS

5.1 None



SI 4.7.C-1 - Secondary Containment Capabilities

6. PROCEDURE

6.1 Initial and perform the indicated sections on the configuration to be tested on Data Sheet SI 4.7.C-1 and perform steps indicated below.

6.1.1 U1RZ - Perform section 6.2 and section 6.4.

6.1.2 U2Rz - Perform section 6.2 and section 6.4.

6.1.3 U3RZ - Perform section 6.2 and section 6.4.

6.1.4 RFZ - Perform section 6.3 and section 6.4.

6.1.5 U1RZ and RFZ - Perform sections 6.2, 6.3 and section 6.4.

6.1.6 U2RZ and RFZ - Perform sections 6.2, 6.3 and section 6.4.

6.1.7 U3RZ and RFZ - Perform sections 6.2, 6.3 and section 6.4.

6.1.8 Reactor Building - Perform section 6.2 for each RZ, and section 6.3 for the RFZ and section 6.4.

6.2 Manual Isolation of a Reactor Zone

NOTE: HS-64-117, HS-64-120, HS-64-119, & HS-64-122 are spring return to normal.

6.2.1 Place HS-64-117 (Panel 9-25) in the TEST position for the reactor zone to be isolated. (Inboard and common valves)

6.2.2 Place HS-64-120 (Panel 9-25) in the TEST position for the reactor zone to be isolated. (Outboard valves)

6.2.3 Start SGTS trains A and B in accordance with OI 65 and verify actions and/or take data as indicated on Data Sheet SI 4.7.C-1 for the reactor zone that is being isolated.

NOTE: If a reactor zone is the only zone to be isolated, go to Section 6.4 and if the refuel zone is to be isolated complete Section 6.3 before going to Section 6.4.

6.3 Manual Isolation of the Plant Refueling Zone

6.3.1 Place HS-64-119 (Panel 9-25-1) in the TEST position. (Inboard and common valves)



SI 4.7.C-1 - Secondary Containment Capabilities

6. PROCEDURE (Continued)

6.3 Manual Isolation of the Plant Refueling Zone (Continued)

6.3.2 Place HS-64-122 (Panel 9-25-1) in the TEST position.

(Outboard valves)

6.3.3 Start SGTS trains A and B, if not already running, in accordance with OI 65 and verify actions and/or take data as indicated on Data Sheet SI 4.7.C-1 for the refueling zone.

6.4 Secondary Containment Capability

NOTE: This section is to verify that the secondary containment is capable of maintaining a  $-1/4"$  H<sub>2</sub>O with a system inleakage of  $\leq 12,000$  cfm for the entire reactor building or allowable in-leakage for one or more zones.

6.4.1 Adjust the total flow indicated on FI-65-50 and FI-65-71 (Panel 9-25-2) to 11,500 cfm for entire reactor building or less than allowable flow for one or more zones as shown in Appendix A by closing a SGTS fan discharge damper or if two fans are running it may be necessary to shut one fan off.

NOTE: Let the flow stabilize for 5 minutes before the next step.

6.4.2 Record flows indicated on FI-65-50 and FI-65-71 (Panel 9-25-2) and total on Data Sheet SI 4.7.C-1.

6.4.3 Record the following static pressures and the air temperatures at the differential pressure indicator locations.

<u>Zone</u>	<u>Instrument</u>	<u>Panel</u>
U1RZ	1-PdIC-64-2	1-25-213
U2RZ	2-PdIC-64-2	2-25-213
U3RZ	3-PdIC-64-2	3-25-213
U1RFZ	1-PdIC-64-1	1-25-219
U2RFZ	2-PdIC-64-1	2-25-215
U3RFZ	3-PdIC-64-1	3-25-215

6.4.4 Verify all four zones static pressures are  $\leq -1/4"$  H<sub>2</sub>O.

SI 4.7.C-1 - Secondary Containment Capabilities

6. PROCEDURE (Continued)

6.4 Secondary Containment Capability (Continued)

6.4.5 Record the wind direction and velocity from the 33 ft. level windspeed.

6.4.5.1 Record air temp. outside reactor building at ground level indicator.

NOTE: If all four zones static pressures are  $\leq -1/4"$  H<sub>2</sub>O this will satisfy 4.7.C.1.b and c. If one or more zones do not meet the  $-1/4"$  H<sub>2</sub>O static pressure requirements the following steps must be completed.

NOTE: Check to assure reactor building is isolated.

6.4.6 Adjust the total flow indicated on FI-65-50 and FI-65-71 (Panel 9-25-2) to 12,000 cfm or allowable flow from Appendix A for one or more zones by opening the fan discharge damper or it may be necessary to start another fan and adjust its discharge damper.

NOTE: Let the flow stabilize for 5 minutes before the next stop. Record flows indicated on FI-65-50 and FI-65-71 (Panel 9-25-2) and total on Data Sheet SI 4.7.C-1.

6.4.7 Record the following static pressures.

<u>Zone</u>	<u>Instrument</u>	<u>Panel</u>
U1RZ	1-PdIC-64-2	1-25-213
U2RZ	2-PdIC-64-2	2-25-213
U3RZ	3-PdIC-64-2	3-25-213
U1RFZ	1-PdIC-64-1	1-25-219
U2RFZ	2-PdIC-64-1	2-25-215
U3RFZ	3-PdIC-64-1	3-25-215

6.4.8 Verify all four zones static pressures are  $\leq -1/4"$  H<sub>2</sub>O.

NOTE: If all four zones static pressures are  $\leq -1/4"$  H<sub>2</sub>O this will satisfy 4.7.C.1.b and c. If one or more zones do not meet the  $-1/4"$  H<sub>2</sub>O static pressure requirement the following steps must be completed.

6.4.9 Isolate the zone(s) that have a static pressure of  $> -1/4"$  H<sub>2</sub>O from the remaining zones and indicate zones isolated on Data Sheet SI 4.7.C-1.

6.4.10 Adjust the total flow indicated on FI-65-50 and FI-65-71 (Panel 9-25-2) to the Allowable Flow of the remaining zones. See Appendix A for the Allowable Flows.

SI 4.7.C-1 - Secondary Containment Capabilities

6. PROCEDURE (Continued)

6.4 Secondary Containment Capability (Continued)

6.4.10 (Continued)

NOTE: Let the flow stabilize for 15 minutes before the next step.

6.4.11 Record flows indicated on FI-65-50 and FI-65-71 (Panel 9-25-2) and total on Data Sheet SI 4.7.C-1.

6.4.12 Record the following static pressures.

<u>Zone</u>	<u>Instrument</u>	<u>Panel</u>
U1RZ	1-PdIC-64-2	1-25-213
U2RZ	2-PdIC-64-2	2-25-213
U3RZ	3-PdIC-64-2	3-25-213
U1RFZ	1-PdIC-64-1	1-25-219
U2RFZ	2-PdIC-64-1	2-25-215
U3RFZ	3-PdIC-64-1	3-25-215

6.4.13 Verify the remaining zones static pressures are  $\leq -1/4"$   $H_2O$ .

NOTE: If these zones static pressures are  $\leq -1/4"$   $H_2O$  this will satisfy 4.7.C-2. Maintenance must be performed on the isolated zone and should be indicated on Data Sheet SI 4.7.C-1 cover sheet under Remarks.

6.4.14 Check those switches that were put in the TEST position under sections 6.2 and/or 6.3 in their NORMAL position and verify on Data Sheet SI 4.7.C-1.

6.4.15 Verify on Data Sheet SI 4.7.C-1 those actions listed under RETURN TO NORMAL for the zones tested.

Appendix A

Secondary Containment Leak Rate Test Criteria

During preoperational testing for unit 3 the following in-leakage rates at  $-1/4''$   $H_2O$  static pressure were documented.

	<u>Documented Inleakage</u>	<u>Percentage</u>
Common refuel floor (units 1, 2, & 3)	6250	64%
Unit 1 reactor zone	1239	12.7%
Unit 2 reactor zone	1064	10.9%
<u>Unit 3 reactor zone</u>	<u>1211</u>	<u>12.4%</u>
Total secondary containment	9764 CFM	100%

Leak rate testing will normally be done with all zones simultaneously lined up for testing; however, plant operating conditions and the technical specification do not always permit this. When less than four zones (3-unit plant) are being tested, the total allowable inleakage will be based on preoperational testing and will be based on the sum of the allowable inleakages for the zones being tested. After a secondary containment violation, the total inleakage must not be greater than the total of the allowable inleakage for each individual zone as indicated below.

The technical specifications require the total secondary containment inleakage to be less than 12,000 cfm at  $-1/4''$  differential pressure. The following are therefore established for surveillance testing various combinations of zones when secondary containment has been violated.

<u>Zone</u>	<u>Allowable Inleakage</u>
Common refuel floor	7680
Unit 1 reactor zone	1524
Unit 2 reactor zone	1308
<u>Unit 3 reactor zone</u>	<u>1488</u>
Reactor Building	<u><math>\leq</math> 12000 CFM</u>

DATA COVER SHEET SI 4.7.C-1

SECONDARY CONTAINMENT CAPABILITY

UNIT 1, 2 OR 3

Performed By Chalmers / Terpstra Date 3/20/78  
Unit Operator

Were criteria satisfied? ☒ Yes ☐ No

If no, notify shift engineer.

If no, was a Limiting Condition for Operation violated? ☐ Yes (explain in remarks)  
☐ No (explain in remarks)

<sup>1</sup>  
Verified by Shift Engineer \_\_\_\_\_, Date \_\_\_\_\_

Reason for test:

\_\_\_\_\_ Maintenance complete on \_\_\_\_\_  
\_\_\_\_\_ Another system ( \_\_\_\_\_ ) inoperable  
☒ Required by schedule  
\_\_\_\_\_ Plant condition (explain) \_\_\_\_\_  
\_\_\_\_\_ Other (explain) \_\_\_\_\_

Results reviewed John E. O'Brien Date 3-20-78  
Assistant Shift Engineer

Results Review and Approval

(1) Cognizant Engineer See App B following sheet Date \_\_\_\_\_

Rescheduled

QA Staff Rum Date 4-3-78

REMARKS Note: Wind Speed 17.1 MPH

(1) Notify Maintenance that inleakage is excessive if criteria are not satisfied.

Data Sheet SI 4.7.C-1 (Continued)

Step	Component/Process Variable	Verify and/or Record	Initials/Date
3.2	Rx. Bldg. Ventilation System	Normal	<u>Sub 3-19-78</u>
3.3	SGTS	Standby Readiness	<u>Sub 3-19-78</u>
3.4	Main Steam Line Tunnel Temperature for units being tested	$\leq 150^{\circ} \text{ F}$	<u>Sub 3-19-78</u> 1
3.5	16A-S34 for units tested	Not in Drywell or Torus Bypass	<u>x Sub 3-19-78</u>
3.8	FI-65-50 UZ 9-25 CR	Currently Calibrated	<u>Rg 3/19/78</u>
	FI-65-71 UZ 9-25 CR	Currently Calibrated	<u>Rg 3/19/78</u>
	PdIC-64-1 (All three units) RFF 25-215	Currently Calibrated	<u>Rg 3/19/78</u>
	PdIC-64-2 (All three units) 25-213	Currently Calibrated	<u>Rg 3/19/78</u>
3.9	Inboard Equipment Access Lock Doors	Closed and Seals Inflated	<u>Sub 3-19-78</u>
3.10	Drywell or Suppression Chamber	Purge Not Inprocess	<u>Sub 3-19-78</u>
3.11	Doors in the following table for the tested zone(s) or the number doors for all zones.	Closed	<u>Sub 3-19-78</u>

4th (su)



Table 3.11  
Data Sheet SI 4.7.C-2 (Continued)

Elevation	U1RZ	U2RZ	U3RZ	RFZ
519.0	30 31	34, 36 35, (37) cone	40 41	
541.5	42	43, 44 cone	45	
565.0	#221 #231 224 #229 #228 #232 #230 #230a	#236 #237 #238 240 242 244 336 U3 air lock U1 air lock	#249 #250 #251 255	255 ✓ 242 ✓ 226 ✓ 231 ✓
583.0	#298	46w 401-11	#826, #827	
593.0	#538 #539 490	497, 501 #540 #541 540	506 #513 #514	
621.25	#534 #637 635	640, 647 #648 #649 649 649 649	651 #657 #658	
639.0	670	672 673	675	46w 401-12
664.0	704 705 706 707	700 701 712 708	713 714 715 716	717 718 719 720
			724 725 722 723	715 716 717 718
				700, 702, 724, 715, 713 701, 703, 725, 716, 714 704, 720, 722, 706, 717 705, 721, 723, 708, 707 712



DATA SHEET SI 4.7.C-1 (Continued)

Page 13  
BF SI 4.7.C  
5/11/76

Step	Component/Process Variable	Verify and/or Record	Initials/Date
6.1.1	U1RZ Configuration		<u>NA</u>
6.1.2	U2RZ Configuration		<u>NA</u>
6.1.3	U3RZ Configuration		<u>NA</u>
6.1.4	RFZ Configuration		<u>NA</u>
6.1.5	U1RZ and RFZ Configuration		<u>NA</u>
6.1.6	U2RZ and RFZ Configuration		<u>APJ 3/17/78</u>
6.1.7	U3RZ and RFZ Configuration		<u>NA</u>
6.1.8	Reactor Building Configuration		<u>NA</u>
6.2.1	1-HS-64-117	TEST Position	<u>NA</u>
	2-HS-64-117	TEST Position	<u>Sub</u> 605
	3-HS-64-117	TEST Position	<u>NA</u>
6.2.2	1-HS-64-120	TEST Position	<u>NA</u>
	2-HS-64-120	TEST Position	<u>*Sub</u>
	3-HS-64-120	TEST Position	<u>NA</u>
6.2.3	SGTS Train A	START	<u>NA</u>
	or SGTS Train B	START	<u>NA</u>
	or SGTS Train C	START	<u>*Sub</u>
<u>Unit 1 Reactor Zone</u>			
	U-1 Reactor Zone Ventilation Sys.	Shutdown	<u>NA</u>
	1-FCO-64-13	Closed	<u> </u>
	1-FCO-64-14	Closed	<u> </u>
	1-FCO-64-42	Closed	<u> </u>
	1-FCO-64-43	Closed	<u>V</u>

Data Sheet SI 4.7.C-1 (Continued)

Step	Component/Process Variable	Verify and/or Record	Initials/Date
6.2.3	(Continued)		
	1-FCO-64-15	Open	NA
	1-FCO-64-40	Open	
	1-FCO-64-41	Open	
	1-FCO-64-60A	Closed	
	1-FCO-64-60B	Closed	
	1-FCO-64-60C	Closed	
	1-FCO-64-60D	Closed	
	U-1 Stair Hall Supply Fan	Shutdown	✓
	<u>Unit 2 Reactor Zone</u>		
	U-2 Reactor Zone Ventilation Sys	Shutdown	x Sub
	2-FCO-64-13	Closed	Sub
	2-FCO-64-14	Closed	Sub
	2-FCO-64-42	Closed	Sub
	2-FCO-64-43	Closed	OS-3 ARC
	2-FCO-64-15	Open	0 ARC
	2-FCO-64-40	Open	0 ARC
	2-FCO-64-41	Open	Sub
	<u>Unit 3 Reactor Zone</u>		
	U-3 Reactor Zone Ventilation Sys	Shutdown	NA
	3-FCO-64-13	Closed	
	3-FCO-64-14	Closed	
	3-FCO-64-42	Closed	
	3-FCO-64-43	Closed	
	3-FCO-64-15	Open	✓
	3-FCO-64-40	Open	✓

## Data Sheet SI 4.7.C-1 (Continued)

Page 15  
BF SI 4.7.C  
5/11/76

Step	Component/Process Variable	Verify and/or Record	Initials/Date
6.2.3	(Continued)		
	<u>Unit 3 Reactor Zone</u> (Continued)		
	3-FCO-64-41	Open	NA
	3-FCO-64-60A	Closed	
	3-FCO-64-60B	Closed	
	3-FCO-64-60C	Closed	
	3-FCO-64-60D	Closed	
	U-3 Stair Hall Supply Fan	Shutdown	W
6.3.1	1-HS-64-119	TEST Position	Sub
6.3.2	1-HS-64-122	TEST Position	Sub
6.3.3	SGTS Train A	START	(X) ARC
	or SGTS Train B	START	N/A
	or SGTS Train C	START	N/A
	<u>Refueling Zone</u>		
	U-1 Area Refueling Zone Vent Sys	Shutdown	Sub
	U-2 Area Refueling Zone Vent Sys	Shutdown	Sub
	U-3 Area Refueling Zone Vent Sys	Shutdown	Sub
	1-FCO-64-6	Close	Sub
	2-FCO-64-6	Close	Sub
	3-FCO-64-6	Close	Sub
	1-FCO-64-5	Close	Sub
	2-FCO-64-5	Close	Sub
	3-FCO-64-5	Close	Sub
	1-FCO-64-10	Close	Sub
	2-FCO-64-10	Close	Sub
	3-FCO-64-10	Close	Sub

Step	Component/Process Variable	Verify and/or Record	Initials/Date
6.3.3	(Continued)		
	<u>Refueling Zone</u> (Continued)		
	1-FCO-64-9	Close	Sub
	2-FCO-64-9	Close	Sub
	3-FCO-64-9	Close	Sub
	1-FCO-64-63	Open	Sub ARC
	1-FCO-64-7	Open	Sub
*	3& 1-FCO-64-65A	Close	JC
*	3& 1-FCO-64-65B	Close	JC
*	3& 1-FCO-64-65C	Close	JC
*	3& 1-FCO-64-65D	Close	JC
	1-FCO-64-44	Open	JC
	1-FCO-64-45	Open	JC
	Equipment Access Lock Exhaust Fan	Shutdown	JC
	U-1 Stair Hall Supply Fan	Shutdown	JC
	U-3 Stair Hall Supply Fan	Shutdown	JC

\* No Equipment Access lock on U-3.

6.4.2 FI-65-50  
FI-65-71  
Total

5200 sfc  
3500 sfc  
8700 sfc

3/19/78  
3/19/78  
3/19/78

Data Sheet SI 4.7.C-1 (Continued)

Step Component/Process Variable

Verify and/or Record Initials/Date

6.4.3	1-PdIC-64- Rx Zone	NA	"H <sub>2</sub> O	NA
	2-PdIC-64- Rx Zone	-0.53	"H <sub>2</sub> O	JS 1/19/78
	3-PdIC-64- Rx Zone	NA	"H <sub>2</sub> O	NA
6.4.3	1-PdIC-64- Refueling Zone	-0.48	"H <sub>2</sub> O	JS 1/19/78
	2-PdIC-64- Refueling Zone	-0.49	"H <sub>2</sub> O	JS 1/19/78
	3-PdIC-64- Refueling Zone	-0.47	"H <sub>2</sub> O	JS 1/19/78
6.4.4	All zones $\leq -1/4"$ H <sub>2</sub> O	SW		JS 1/19/78
6.4.5	XR-90-102	Direction		JS
		17.1	mph	JS
6.4.6	FI-65-50		scfm	NA
	FI-65-71		scfm	
	Total		scfm	
6.4.7	1-PdIC-64-2 Rx Zone		"H <sub>2</sub> O	
	2-PdIC-64-2 Rx Zone		"H <sub>2</sub> O	
	3-PdIC-64-1 Rx Zone		"H <sub>2</sub> O	
	1-PdIC-64-1 Refueling Zone		"H <sub>2</sub> O	
	2-PdIC-64-1 Refueling Zone		"H <sub>2</sub> O	
	3-PdIC-64-1 Refueling Zone		"H <sub>2</sub> O	
6.4.8	All zones $\leq -1/4"$ H <sub>2</sub> O			
6.4.9	Zones Isolated	List		

<u>Step</u>	<u>Component/Process Variable</u>	<u>Verify and/or Record</u>	<u>Initials/Date</u>
	<u>Return to Normal</u>		
6.4.11	FI-65-50	_____ scfm	NA
	FI-65-71	_____ scfm	
	Total	_____ scfm	
6.4.12	1-PdIC-64-2 Rx Zone	_____ "H <sub>2</sub> O	
	2-PdIC-64-2 Rx Zone	_____ "H <sub>2</sub> O	
	3-PdIC-64-2 Rx Zone	_____ "H <sub>2</sub> O	
	1-PdIC-64-1 Refueling Zone	_____ "H <sub>2</sub> O	
	2-PdIC-64-1 Refueling Zone	_____ "H <sub>2</sub> O	
	3-PdIC-64-1 Refueling Zone	_____ "H <sub>2</sub> O	
6.4.13	Remaining zones $\leq -1/4$ "H <sub>2</sub> O		
6.4.14	1-HS-64-117	NORMAL	NA
	2-HS-64-117	NORMAL	(S) ARC
	3-HS-64-117	NORMAL	NA
	1-HS-64-120	NORMAL	NA
	2-HS-64-120	NORMAL	(W) ARC
	3-HS-64-120	NORMAL	NA
	1-HS-64-119	NORMAL	+ ARC
	1-HS-64-122	NORMAL	(*) ARC



<u>Step</u>	<u>Component/Process Variable</u>	<u>Verify and/or Record</u>	<u>Initials/Date</u>
	<u>Return to Normal</u>		
6.4.15	SGTS Fan A	OFF	ARC
	FCO-65-3	Closed	ARC
	A-SGTS humidity control heater	OFF	ARC
	A-SGTS charcoal heater	ON	ARC
	SGTS Fan B	OFF	NA
	FCO-65-25	Closed	NA
	B-SGTS humidity control heater	OFF	NA
	B-SGTS charcoal heater	ON	NA
	SGTS Fan C	OFF	NA
	FCO-65-51	Closed	NA
	C-SGTS humidity control heater	OFF	NA
	C-SGTS charcoal heater	ON	NA

Unit 1 Reactor Zone

U-1 Reactor Zone Ventilation System	In operation as required	
1-FCO-64-13	OPEN	NA
1-FCO-64-14	OPEN	
1-FCO-64-42	OPEN	
1-FCO-64-43	OPEN	
1-FCO-64-15	Closed	
1-FCO-64-40	Closed	
1-FCO-64-41	Closed	
1-FCO-64-60A	Open	
1-FCO-64-60B	Open	
1-FCO-64-60C	Open	
1-FCO-64-60D	Open	
U-1 Stair Hall Supply Fan	In operation as required	



<u>Step</u>	<u>Component/Process Variable</u>	<u>Verify and/or Record</u>	<u>Initials/Date</u>
6.4.15	(Continued)		
	<u>Unit 2 Reactor Zone</u>		
	U-2 Reactor Zone Ventilation System	In operation as required	
	2-FCO-64-13	Open	<u>ARC</u>
	2-FCO-64-14	Open	<u>ARC</u>
	2-FCO-64-42	Open	<u>ARC</u>
	2-FCO-64-43	Open	<u>ARC</u>
	2-FCO-64-15	Closed	<u>ARC</u>
	2-FCO-64-40	Closed	<u>ARC</u>
	2-FCO-64-41	Closed	<u>ARC</u>
	<u>Unit 3 Reactor Zone</u>		
	U-3 Reactor Zone Ventilation System	In operation as required	
	3-FCO-64-13	Open	<u>NA</u>
	3-FCO-64-14	Open	
	3-FCO-64-42	Open	
	3-FCO-64-43	Open	
	3-FCO-64-15	Closed	
	3-FCO-64-40	Closed	
	3-FCO-64-41	Closed	
	3-FCO-64-60A	Open	
	3-FCO-64-60B	Open	
	3-FCO-64-60C	Open	
	3-FCO-64-60D	Open	
	U-3 Stair Hall Supply Fan	In Operation as Required	

## Data Sheet SI 4.7.C-1(Continued)

Page 21  
BF SI 4.7.C  
5/11/76

Step	Component/Process Variable	Verify and/or Record	Initials/Date
6.4.15	(Continued)		
	<u>Refueling Zone</u>		
	U-1 Area Refueling Zone Vent Sys	In Operation as required	<u>ARC</u>
	U-2 Area Refueling Zone Vent Sys	In Operation as required	<u>ARC</u>
	U-3 Area Refueling Zone Vent Sys	In Operation as required	<u>ARC</u>
	1-FCO-64-6	Open	<u>ARC</u>
	2-FCO-64-6	Open	<u>ARC</u>
	3-FCO-65-6	Open	<u>ARC</u>
	1-FCO-64-5	Open	<u>ARC</u>
	2-FCO-64-5	Open	<u>ARC</u>
	3-FCO-64-5	Open	<u>ARC</u>
	1-FCO-64-10	Open	<u>ARC</u>
	2-FCO-64-10	Open	<u>ARC</u>
	3-FCO-64-10	Open	<u>ARC</u>
	1-FCO-64-9	Open	<u>ARC</u> TC
	2-FCO-64-9	Open	<u>ARC</u>
	3-FCO-64-9	Open	<u>ARC</u>
	1-FCO-64-63	Close	<u>ARC</u>
	1-FCO-64-7	Close	<u>ARC</u>
	3& 1-FCO-64-65A	Open	<u>IL</u>
	3& 1-FCO-64-65B	Open	<u>IL</u>
	3& 1-FCO-64-65C	Open	<u>IL</u>
	3& 1-FCO-64-65D	Open	<u>IL</u>
	1-FCO-64-44	Close	<u>ARC</u>
	1-FCO-64-45	Close	<u>ARC</u>
	Equipment Access Lock Exhaust Fan	In Operation as required	<u>IL</u>
	U-1 Stair Hall Supply Fan	In Operation as required	<u>ARC</u>
	U-3 Stair Hall Supply Fan	In Operation as required	<u>IL</u>

## Unit

FROM: LFB

DATE: 3-20-78

SI 4.7.C-1, SCLRT

Date 3-20-77 has deficiencies on the data sheet(s) as described below.  
Performed 1038

Followup data reviewed and approved

Cognizant Engineer

Date 3-29-77

- (1) Corrective action must be filled out explicitly, addition sheets or the end of the form may be attached, or references to documents, such as maintenance history records or trouble reports, that contain explicit information so that common-mode failures can be evaluated and quality assurance established.
- (2) Signature and date of person resolving deficiency required for each item.

Attachment to SI 4.7.C.1 (Unit 2/3-20-78)

Calculate dynamic pressure,  $q$ :

(See FSAR § 12.2.2.9.1)  $q = .002552 V^2$

$$q = .002552 (17.1)^2 \text{ psf} \left( \frac{.01602 \text{ ft H}_2\text{O}}{\text{psf}} \right) \left( \frac{12 \text{ in H}_2\text{O}}{\text{ft H}_2\text{O}} \right)$$

$$\boxed{q = .14 \text{ in H}_2\text{O}}$$

Note: Containment  $\Delta P_{\text{res}} .5$

Limit = .25

Margin = .25 in H<sub>2</sub>O

Analysis

	Unit 2	Refuel Zone	Total
Limit (In House)	1308	+ 7680	= 8988
Measured	3500	+ 5200	= 8700

Margin to Limit =  $\boxed{288 \text{ scfm}}$

On 3-20-78, a margin to the leak rate limit of 288 scfm was measured for unit 2. This margin would be reduced at calm wind conditions. However, since the dynamic pressure of the winds ~~at~~ (.14 in H<sub>2</sub>O) was less than the excess of  $\Delta p$  (.25 in H<sub>2</sub>O) occurring during the test, there was still sufficient conservatism to satisfy Tech Spec require-

Prepared By  
The Man.

Look Darling, See  
That I Get His back!

L.D.I.