



Northern States Power Company

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February 21, 1985

U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket No. 50-282 License No. DPR-42

One Containment Isolation Valve Failed Its
Local Leakage Rate Test

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72
on January 23, 1985.

David Musolf
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

8503040494 850221
PDR ADOCK 05000282 PDR
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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Prairie Island Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 8 2										PAGE (3) 1 OF 0 2																																							
TITLE (4) One Containment Isolation Valve Failed Its Local Leakage Rate Test																																																											
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																													
MONTH			DAY			YEAR				YEAR			SEQUENTIAL NUMBER			REVISION NUMBER				MONTH			DAY			YEAR				FACILITY NAMES										DOCKET NUMBER(S)																			
0 1			2 3			8 5				8 5			0 0 5			0 0				0 2			2 1			8 5														0 5 0 0 0																			
0 1			2 3			8 5				8 5			0 0 5			0 0				0 2			2 1			8 5														0 5 0 0 0																			
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																																	
POWER LEVEL (10)										20.402(b)										20.406(c)										50.73(a)(2)(iv)										73.71(b)																			
										20.406(a)(1)(i)										50.38(e)(1)										50.73(a)(2)(v)										73.71(e)																			
										20.406(a)(1)(ii)										50.38(c)(2)										50.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 365A)																			
										20.406(a)(1)(iii)										X 50.73(a)(2)(i)										50.73(a)(2)(vii)(A)																													
										20.406(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(vii)(B)																													
										20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(v)																													
LICENSEE CONTACT FOR THIS LER (12)																																																											
NAME Arne A Hunstad, Staff Engineer																				TELEPHONE NUMBER 6 1 2 3 8 8 - 1 1 2 1																																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																											
CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NPROS					CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NPROS														
SUPPLEMENTAL REPORT EXPECTED (14)																																																											
X YES (If yes, complete EXPECTED SUBMISSION DATE)																				NO										EXPECTED SUBMISSION DATE (15)																													
																														0 3 2 9 8 5																													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During local leakage rate testing at refueling, one containment isolation check valve exhibited leakage in excess of Tech Spec limits. The valve was replaced with a spare. A supplemental report will be submitted when the cause of the failure is determined.

DPE 8502270478

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Prairie Island Unit 1	0 5 0 0 0	8 5	— 0 0 5	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During local leakage rate testing at refueling, one containment isolation check valve exhibited leakage in excess of Tech Spec limits. The valve was replaced with a spare. A supplemental report will be submitted when the cause of the failure is determined.