

NRC Form 366  
(9-83)U.S. NUCLEAR REGULATORY COMMISSION  
APPROVED OMB NO. 3150-0104  
EXPIRES 8/31/85

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>SURRY POWER STATION, UNIT 1</b>										DOCKET NUMBER (2) <b>0 5 0 0 0 2 8 p</b>				PAGE (3) <b>1 OF 0 3</b>	
TITLE (4) <b>LOSS OF ADMINISTRATIVE CONTROL</b>															
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)		
0 1	0 1	8 5	8 5	0 0 1	0 0 0 1	0 1	2 8	8 5					0 5 0 0 0		
														0 5 0 0 0	
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)													
N		20.402(a)				20.405(c)				50.73(a)(2)(iv)				73.71(b)	
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)	
0 3 4		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12)															
NAME										TELEPHONE NUMBER					
R. F. SAUNDERS, STATION MANAGER										AREA CODE 8 0 4 3 5 7 - 3 1 8 8					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR	
YES (If yes, complete EXPECTED SUBMISSION DATE)										NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On January 1, 1985 with the unit at 34% power, at 0100 hours, it was discovered that a safe signal was in place in "B" loop channel 2 of overtemperature delta T (OPΔT) without the cognizance of the Operations or Instrumentation staff. Safe signals are permitted by Technical Specifications for short periods of time during testing, however, this does imply cognizance of the signal.

With the activity associated with the trip on 12-31-84 and the subsequent investigations to determine the trip cause, Admin. Control of the safe signal was not maintained in the turnover from day shift to swing shift.

A procedure was written that will be used when a safe signal is required. Details of the event will be reviewed by licensed personnel. The NI's were rescaled, the "B" loop ΔT channel was placed in trip and the safe signal removed.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
SURRY POWER STATION, UNIT 1	0 5 0 0 0 2 8 0	8 5	— 0 0 1	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

1. Description of the Event

On January 1, 1985 with the unit at 34% power, at 0100 hours, it was discovered that a safe signal was in place in "B" loop channel 2 of over-temperature delta T (OTΔT) without the cognizance of the Operations or Instrumentation staff. Safe signals are permitted by Technical Specifications for short periods of time during testing, however, this does imply cognizance of the signal.

This event began on 12-29-84 when the (OTΔT) channel 2 bistables for "B" loop were placed in trip due to a failed RTD. On 12-31-84 at 1147 hours, a safe signal was injected, with the permission of the Shift supervisor, into the "B" loop ΔT instrumentation channel to allow performance of Periodic Test 8.1 (Reactor Protection Logic) and to rescale the excore nuclear detectors (NI's). During this test, a reactor trip occurred at 1324 hours (See Unit 1 LER-84-026-00). The test was terminated, but the safe signal remained in for rescaling the NI's. This information was not passed on to the subsequent shifts and was not detected by pre-criticality checklists.

2. Probable Consequences

The OTΔT logic is designed to protect the core against departure from nucleate boiling (DNB). Sufficient margin to DNB is maintained with the unit at full power. A very large margin to DNB was maintained from the reactor trip until the safe signal was discovered because reactor power remained below 35%.

The total time the safe signal was in place was 11 hours 13 minutes, however, the time that plant personnel were unaware of it was 5 hours 30 minutes. Also, the 2 redundant OTΔT channels remained operable throughout this event, therefore an unreviewed safety question was not created and the health and safety of the public were unaffected.

3. Cause

With the activity associated with the trip on 12-31-84 and the subsequent investigations to determine the trip cause, Admin. Control of the safe signal was not maintained in the turnover from day shift to swing shift. This information was available in the Control Room Operator's log.

4. Immediate Corrective Action

When the safe signal was discovered, administrative control was reestablished, and since it was still needed for rescaling the NI's, no immediate corrective actions were taken.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

5. Additional Corrective Actions

The NI's were rescaled, and the "B" loop  $\Delta T$  channel was placed in trip and the safe signal removed.

6. Action Taken to Prevent Recurrence

A procedure was written that will be used when a safe signal is required. Details of the event will be reviewed by licensed personnel.

7. Generic Implications

None.



VIRGINIA ELECTRIC AND POWER COMPANY  
Surry Power Station  
P. O. Box 315  
Surry, Virginia 23883

January 28, 1985

Serial No: 85-001

Docket No: 50-280

License No: DPR-32

U. S. Nuclear Regulatory Commission  
Document Control Desk  
016 Phillips Building  
Washington, D.C. 20555

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

REPORT NUMBER

85-001-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

R. F. Saunders  
Station Manager

Enclosure

cc: Mr. James P. O'Reilly  
Regional Administrator  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30323

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