

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Beaver Valley Power Station, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 3 4				PAGE (3) 1 OF 0 3				
TITLE (4) Main Steam Safety Valve Lift Settings Outside Allowable Limits																		
EVENT DATE (6)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)					
1	0	1	2	8	4	8	4	0	1	1	0	1	1	1	8	5	N/A	0 5 0 0 0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																
3		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)				
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)				
0 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)				
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)								
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)								
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)								
LICENSEE CONTACT FOR THIS LER (12)																		
NAME Robert J. Druga, Manager - Technical Services										TELEPHONE NUMBER 4 1 2 6 4 3 - 5 3 0 8								
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																		
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS								
X	S	B R V	D	2 4 5	Y													
SUPPLEMENTAL REPORT EXPECTED (14)																		
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO		EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/12/84, while in Hot Standby, lift settings on eight (8) of the fifteen (15) main steam safety valves were found outside the Technical Specification 3.7.1.1 operating limits of plus or minus 1%. Of the eight valves, one had a lift setting slightly below the normal operating limit, while seven (7) had lift settings above their normal operating limits. The valves were all adjusted to within the allowable limits and tested satisfactorily. The root cause for this incident has been attributed to setpoint drift. The valves are Type 3707 RAX-RT21, manufactured by the Dresser Valve Division. As a result of this incident, one failed main steam safety valve was sent to Wyle Laboratories for overhaul and possible repair. This valve tested satisfactorily at Wyle; however, the valve disc was replaced based on a recommendation from the Dresser representative. This valve was then re-installed at Beaver Valley. A portion of the main steam safety valves were again tested during the plant startup from the current refueling outage. The valves chosen for testing were the five (5) valves from the 10/12/84 test in which the lift setpoints could not be determined. During this test, two valves exceeded their operational limits. The cause was again attributed to setpoint drift. These two valves were adjusted to within their operational limits and returned to service.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)			
		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER						
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 10/12/84, while in Hot Standby lift settings on eight (8) of the fifteen (15) main steam safety valves were found outside the Technical Specification 3.7.1.1 operating limits of plus or minus 1%. The mark numbers, required lift settings, as-found lift pressure settings and the as-left lift pressure settings were as follows:

MARK NUMBER	REQUIRED SETTING	AS-FOUND SETTING	AS-LEFT SETTING
SV-MS-101A	1075	1078.63	1078.63
SV-MS-101B	1075	1061.73	1070.42
SV-MS-101C	1075	1083.20	1083.20
SV-MS-102A	1085	1086.06	1086.06
SV-MS-102B	1085	1085.39	1085.39
SV-MS-102C	1085	1106.63	1077.46
SV-MS-103A	1095	> 1121.10	1095.52
SV-MS-103B	1095	1106.82	1099.51
SV-MS-103C	1095	> 1133.43	1105.01
SV-MS-104A	1110	> 1137.10	1103.41
SV-MS-104B	1110	1120.03	1120.03
SV-MS-104C	1110	> 1135.77	1116.26
SV-MS-105A	1125	> 1152.10	1118.84
SV-MS-105B	1125	1126.37	1126.37
SV-MS-105C	1125	1135.12	1135.12

All settings are psig.

The above listed valves were all adjusted to the As-Left settings shown above and tested satisfactorily. The root cause for this incident has been attributed to setpoint drift. The valves are Type 3707 RAX-RT21, manufactured by the Dresser Valve Division

As a result of this incident, one failed main steam safety valve (SV-MS-104A) was sent to Wyle Laboratories for overhaul and possible repair. This valve tested satisfactorily at Wyle Laboratories; however, the valve disc was replaced at this time based on a recommendation from the Dresser representative present at Wyle. The disc and body seating surfaces also looked good. No corrosion or steam cutting was found. The Dresser representative judged the valve internals to be in very good condition. This valve was then returned to Beaver Valley and assembled in its proper position in the 1A main steamline.

A portion of the main steam safety valves were again tested during the plant startup from the current refueling outage on 12/28/84. Five main steam safety valves were selected for testing. The five valves chosen for testing on 12/28/84 were chosen because the lift pressure setpoints for these valves could not be determined on the 10/12/84 test. During the 10/12/84 test, these valves exceeded the capability of the test equipment to indicate the setpoint. The mark numbers,

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U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

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		0 1 1	0 1	0 3	of	0 3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

the 10/12/84 As-Left lift pressure setpoints, the 12/28/84 As-Found lift pressure setpoints, and the 12/28/84 As-Left lift pressure setpoints were as follows:

<u>MARK NUMBER</u>	<u>10/12/84 AS- LEFT SETPOINTS</u>	<u>12/28/84 AS- FOUND SETPOINTS</u>	<u>12/28/84 AS- LEFT SETPOINTS</u>
SV-MS-103A	1095.5	1094	1094
SV-MS-103C	1105.01	1114.03	1088
SV-MS-104A	1103.41	1112	1107.6
SV-MS-104C	1116.26	1103	1103
SV-MS-105A	1118.84	1107.79	1118

All settings were psig.

Two of these valves failed to meet the operating limits of plus or minus 1% (SV-MS-103C and SV-MS-105A). SV-MS-103C failed high at 1.74% (1114.03 psig). SV-MS-105A failed low at 1.53% (1107.79 psig). These valves were adjusted to within their operating limits and returned to service. The cause for these failures has again been attributed to setpoint drift.

Presently the Licensing and Compliance Group is investigating the Main Steam Valve Technical Specification and the Bases behind this Technical Specification. Conversations with the vendor and investigations into ASME Section XI requirements and the Power Test Codes will result in submission of a Technical Specification Change.

This is the fourth Licensee Event Report issued for this type of incident.



Nuclear Division
P. O. Box 4
Shippingport, PA 15077-0004

Telephone (412) 393-6000

January 11, 1985
ND1SS1 2320

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
LER 84-011-01

Dr. Thomas E. Marley
Regional Administrator
United States Nuclear Regulatory Commission
Region I
Park Avenue
King of Prussia, PA 19046

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 84-011-01, 10 CFR 50.73.a.2.ii, "Degradation of Safety Valve Operability".

Very truly yours,


Wm. S. Lacey
Plant Manager

md

Attachment

IE22
1/1

T. E. Murley
January 11, 1985
NDISS1:2320
Page two

cc: Director of Management & Program Analysis
United States Nuclear Regulatory Commission
Washington, D.C. 20555

C. A. Roteck, Ohio Edison

Director, Office of Inspection and Enforcement Headquarters
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Peter Tam, BVPS Licensing Project Manager
United States Nuclear Regulatory Commission
Washington, D.C. 20555

W. Troskoski, Nuclear Regulatory Commission, BVPS Site Inspector

Mr. Alex Timme, CAPCO Nuclear Projects Coordinator, Toledo Edison

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

G. E. Muckle, Factory Mutual Engineering, Pittsburgh

Mr. J. A. Triggiani, Operating Plant Projects Manager
Mid Atlantic Area
Westinghouse Electric Corporation
Nuclear Services Integration Division
Box 2728
Pittsburgh, PA 15230

American Nuclear Insurers
c/o Dottie Sherman, ANI Library
The Exchange Suite 245
270 Farmington Avenue
Farmington, CN 06032