

**LICENSEE EVENT REPORT (LER)**

APPROVED OMB NO. 2160-0104  
EXPIRES - 8/31/85

FACILITY NAME (1) <b>Limerick Generating Station - Unit 1</b>										DOCKET NUMBER (2) <b>0 5 0 0 0 3 5 2</b>					PAGE (3) <b>1 OF 0</b>					
TITLE (4) <b>Failure to Demonstrate Adequate Shutdown Cooling</b>																				
EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME			DOCKET NUMBER(S)								
1	2	29	8	4	8	4	0	4	4	0	0	0	1	2	5	8	5	0 5 0 0 0 1 1		
OPERATING MODE (9) <b>4</b>			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 18 CFR §. (Check one or more of the following) (11)																	
POWER LEVEL (10) <b>0 0 0</b>			20.402(b)			20.406(a)			60.73(a)(2)(v)			73.71(b)								
			20.406(a)(1)(ii)			60.73(a)(1)			60.73(a)(2)(v)			73.71(a)								
			20.406(a)(1)(iv)			60.73(a)(2)			60.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Test, NRC Form 364A)								
			20.406(a)(1)(iii)			60.73(a)(2)(ii)			60.73(a)(2)(viii)(A)											
			20.406(a)(1)(iv)			60.73(a)(2)(iii)			60.73(a)(2)(viii)(B)											
			20.406(a)(1)(v)			60.73(a)(2)(iv)			60.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																				
NAME <b>John C. Nagle, Engineer - Special Projects</b>										TELEPHONE NUMBER AREA CODE <b>2 1 5</b> <b>8 4 1 - 5 1 8 4</b>										
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																				
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (16)		MONTH	DAY	YEAR						
YES (If yes, complete EXPECTED SUBMISSION DATE)										NO										

ABSTRACT (LIMIT IS 1400 WORDS, I.E., APPROXIMATELY FIFTEEN SINGLE-SPACE TYPEWRITTEN LINES) (18)

Abstract: 84-044

On December 29, 1984, with Unit 1 in cold shutdown, the 'A' residual heat removal system shutdown cooling loop was blocked out-of-service for maintenance. Technical Specifications require demonstrating an alternate method of decay heat removal within one hour of declaring a shutdown cooling loop inoperable while in cold shutdown. The alternate method was not demonstrated as required, resulting in a failure to meet an action statement of the Technical Specifications. Immediately after discovery of this oversight, the 'A' loop of shutdown cooling was unblocked and returned to operable status.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  Limerick Generating Station Unit 1	DOCKET NUMBER (2)  0 5 0 0 0 3 5 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 4 4	- 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Description of the Event:

On December 29, 1984, at 11:30 a.m. a failure to comply with an action statement of the Technical Specifications occurred. Technical Specification 3.4.9.2 requires that two shutdown cooling loops of the Residual Heat Removal system shall be operable and, unless at least one recirculation pump is in operation, at least one shutdown cooling loop shall be in operation. This Technical Specification is applicable in Operating Condition 4 (cold shutdown).

At the time of the event, the 'B' loop of shutdown cooling was in service. Both reactor recirculation pumps were out-of-service. The 'A' loop shutdown cooling return valve, HV-51-1F015A, was blocked out-of-service for maintenance, thereby rendering this loop inoperable. The action statement for Technical Specification 3.4.9.2 requires that, with less than the required number of Residual Heat Removal shutdown cooling loops operable, within one hour and at least twenty-four hours thereafter, demonstrate the operability of at least one alternate method capable of decay heat removal for each inoperable RHR shutdown cooling loop. This action statement was not met.

Consequences of the Event:

At the time of the event, there was virtually no decay heat in the reactor since the reactor had attained initial criticality on December 22, 1984, and had been operated at very low power levels for short periods of time. Consequently, there were no adverse effects.

Cause of the Event:

Cause of the event was personnel error. The Shift Supervisor who gave permission to apply the block to HV-51-1F015A did not realize that it was necessary to demonstrate an alternate method of decay heat removal, under the existing plant conditions, within one hour of blocking the 'A' loop out-of-service.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Corrective Actions:

The block on the 'A' loop RHR shutdown cooling valve, HV-51-1F015A, was cleared immediately after realizing that the action statement for Technical Specification 3.4.9.2 had not been performed. Clearing the block on HV-51-1F015A returned the 'A' loop of shutdown cooling to operability, thereby satisfying Technical Specification 3.4.9.2.

Additionally, the Shift Supervisor who signed approval for applying the block was orally counseled concerning the event.

PHILADELPHIA ELECTRIC COMPANY

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January 25, 1985

Docket No. 50-352

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Washington, DC 20555

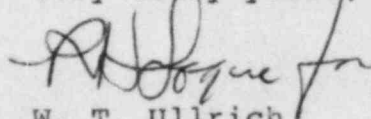
SUBJECT: Licensee Event Report  
Limerick Generating Station - Unit 1

This LER deals with failure to comply with an action statement of shutdown cooling Technical Specification 3.4.9.2.

Reference:	Docket No. 50-352
Report Number:	84-044
Revision Number:	00
Event Date:	December 29, 1984
Report Date:	January 25, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC  
J. T. Wiggins, Senior Site Inspector  
See Service List

1522  
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cc: Judge Helen F. Hoyt  
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Atomic Safety & Licensing Board Panel  
Docket & Service Section (3 Copies)  
James Wiggins  
Timothy R. S. Campbell

January 16, 1985