

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 8 3				PAGE (3) 1 OF 0 2								
TITLE (4) Inadvertent Engineered Safety Features Actuations																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)									
1	2	3	0	8	4	8	4	0	6	7	0	0	0	1	2	5	8	5	0 5 0 0 0 0			
OPERATING MODE (9) 1			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																			
POWER LEVEL (10) 11010			20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)							
			20.405(a)(1)(i)				50.36(c)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)							
			20.405(a)(1)(ii)				50.36(c)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
			20.405(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)											
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)											
			20.405(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																						
NAME Charles D. Naslund - Superintendent, I&C										TELEPHONE NUMBER AREA CODE 3114 61761-18151010												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR						
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO										

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 12/30/84 and 1/9/85 Containment Purge Isolations and Control Room Ventilation Isolations occurred due to downscale (loss of power) trips from a containment atmosphere radiation monitor. The required Engineered Safety Features (ESF) equipment performed as designed when actuated.

In both incidents I&C technicians were contacted to investigate the loss of power. On 12/30/84 the investigation failed to identify the cause of the failure. The monitor was determined operable and returned to service. On 1/9/85 the investigation revealed that a fuse clip to the microprocessor for the monitor was not making adequate contact with the fuse. The fuse clip was tightened and the monitor returned to service. No further corrective action is deemed necessary.

There was no damage to plant equipment or release of radioactivity as a result of these incidents. The ESF actuations were initiated by downscale trips and not in response to high radiation levels, therefore the public health and safety was not endangered during these events.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

Callaway Plant Unit 1

0 5 0 0 0 4 8 3 8 4 -- 0 6 7 -- 0 0 0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On 12/30/84 and 1/9/85 Engineered Safety Features (ESF) were actuated as a result of downscale (loss of power) trips from containment atmosphere radiation monitor GT-RE-32. The downscale trips resulted in the "fail-safe" actuations of Containment Purge Isolations (CPIS) and Control Room Ventilation Isolations (CRVIS). The required ESF equipment performed as designed.

On 12/30/84 the plant was in Mode 1 and at 100% Reactor Power. At approximately 0813 CST a CPIS and CRVIS occurred due to a downscale trip from GT-RE-32. The operators verified proper functioning of the CPIS and CRVIS per plant operating procedures and declared the monitor inoperable.

I&C technicians contacted to investigate the failure of GT-RE-32 found that the microprocessor, RM-80, for the monitor was not functioning due to an apparent loss of power. During the investigation opening and closing of the cabinet door to the RM-80 and tapping on the power supply caused a loss of power to the monitor. The investigation was stopped to obtain the proper work authorization to troubleshoot/repair the power supply. When troubleshooting of the power supply resumed, the failure condition could not be repeated. Additional investigation showed the monitor to be operable and at 1545 it was returned to service.

At 1958 on 1/9/85 a CPIS and CRVIS again occurred due to a power failure to GT-RE-32. The plant was in Mode 1 and at 48% Reactor Power at the time of the actuations. The operators verified proper functioning of the CPIS and CRVIS and took the monitor out of service.

Upon investigation I&C technicians found that the fuse clip which holds the fuse to the RM-80 was loose thus allowing the fuse to make poor electrical contact. The fuse clip was tightened and the monitor returned to service at approximately 0530 on 1/10/85.

The loose fuse clip explains the unidentified cause of the downscale trip on 12/30/84 and is also believed to be the cause of the blown fuse which caused the downscale trip of GT-RE-32 on 12/7/84. The resultant ESF actuations on 12/7/84 were reported in LER 84-062-00 on 1/4/85. The loose fuse clip is considered a single random failure for which adequate corrective action has been provided. Therefore no further corrective action is deemed necessary.

There was no damage to plant equipment or release of radioactivity as a result of these incidents. Since these events were initiated by downscale trips of GT-RE-32 and not in response to high radiation levels, the public health and safety was not threatened at any time during the course of the events.

Previous occurrence: LER 84-062-00

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
P. O. BOX 620
FULTON, MO. 65251

January 25, 1985

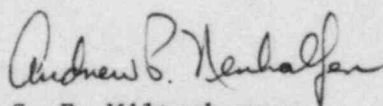
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ULNRC-1025

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-067-00
INADVERTENT ENGINEERED SAFETY FEATURES ACTUATIONS

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning inadvertent Engineered Safety Features actuations due to a loss of power to a radiation monitor.


S. E. Miltenberger
for Manager, Callaway Plant

CDN/WRR/JWK/drs
Enclosure

cc: Distribution attached

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cc distribution for ULNRC-1025

Mr. James G. Keppler
Regional Administrator
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

American Nuclear Insurers
c/o Dottie Sherman, Library
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270 Farmington Avenue
Farmington, CT 06032

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

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