

LICENSEE EVENT REPORT (LER)

APPROVED ONE NO. 3150-0104
EXPIRES - 8/31/85FACILITY NAME (1)
Limerick Generating Station - Unit 1DOCKET NUMBER (2)
0 5 0 0 0 3 5 2 1 OF 0 3

TITLE (4)

RHR Shutdown Cooling Isolation

EVENT DATE (6)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER (5)	
11	26	84	84	019		12	26	84		0 5 0 0 0 1 1	
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OPERATING MODE (3)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																														
4	<table border="1"><tr><td>20.402(a)</td><td>20.408(a)</td><td>X</td><td>30.736(12)(a)</td><td>72.716(a)</td></tr><tr><td>20.406(a)(1)(i)</td><td>30.34(a)(1)</td><td></td><td>30.736(12)(b)</td><td>72.716(b)</td></tr><tr><td>20.406(a)(1)(ii)</td><td>30.34(a)(2)</td><td></td><td>30.736(12)(c)</td><td>OTHER (Specify in Abstract below and in Test, NRC Form 366A)</td></tr><tr><td>20.406(a)(1)(iii)</td><td>30.736(12)(d)</td><td></td><td>30.736(12)(e)(i)(A)</td><td></td></tr><tr><td>20.406(a)(1)(iv)</td><td>30.736(12)(e)</td><td></td><td>30.736(12)(e)(ii)(B)</td><td></td></tr><tr><td>20.406(a)(1)(v)</td><td>30.736(12)(f)</td><td></td><td>30.736(12)(f)</td><td></td></tr></table>	20.402(a)	20.408(a)	X	30.736(12)(a)	72.716(a)	20.406(a)(1)(i)	30.34(a)(1)		30.736(12)(b)	72.716(b)	20.406(a)(1)(ii)	30.34(a)(2)		30.736(12)(c)	OTHER (Specify in Abstract below and in Test, NRC Form 366A)	20.406(a)(1)(iii)	30.736(12)(d)		30.736(12)(e)(i)(A)		20.406(a)(1)(iv)	30.736(12)(e)		30.736(12)(e)(ii)(B)		20.406(a)(1)(v)	30.736(12)(f)		30.736(12)(f)	
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20.406(a)(1)(v)	30.736(12)(f)		30.736(12)(f)																												

LICENSEE CONTACT FOR THIS LER (13)
NAME
B. L. Clark, Senior Engineer-Special ProjectsTELEPHONE NUMBER
AREA CODE
2 1 5 8 4 4 - 5 0 4 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (14)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (15)		EXPECTED SUBMISSION DATE (16)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	NO				

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-spaced typewritten lines) (17)

Abstract: 84-019

On November 26, 1984, the inboard RHR shutdown cooling suction valve closed as the result of receiving an isolation signal. The isolation was caused by an inadvertent actuation of one of the reactor steam dome high pressure switches. After the isolation signal cleared, the RHR system was returned to normal operation.

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11/22/84

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED ONS NO. 3150-0194
EXPIRES 8/31/85

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR 84	SEQUENTIAL NUMBER 0 1 9	REVISION NUMBER 0 0			
					0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 368A (17))

Description of the Event:

On November 26, 1984, at 3:23 a.m., prior to initial criticality, the RHR shutdown cooling inboard isolation valve HV-51-1F009 closed to the isolation position as the result of receiving a spurious reactor high pressure isolation signal. The RHR pumps that were running to facilitate reactor vessel heatup for the vessel hydrostatic test tripped as a result of the closure of the suction valve. After the isolation signal cleared the shutdown cooling mode of RHR was returned to service.

Consequences of the Event:

The nuclear steam supply shutoff system operated properly by isolating the RHR shutdown cooling system as a result of the reactor high pressure signal. During the time that RHR shutdown cooling was out of service, reactor water cleanup and control rod drive cooling were in operation to maintain cooling circulation to satisfy Technical Specification 3.4.9.2. There were no adverse consequences due to the isolation, since there was no decay heat being produced at the time of the event.

Cause of the Event:

Cause of the event is believed to be actuation of one of the two reactor steam dome high pressure instruments that are capable of closing the inboard RHR shutdown cooling suction valve. These devices are PT-42-INO78A and PT-42-INO78B. At the time of the event, Instrument Technicians were backfilling instrument legs on rack C004. This rack contains PT-42-INO78A. At the same time, a response time test was being performed on PT-42-INO78B. A spurious signal from either of these transmitters causes their associated trip units to actuate. A light on the trip unit indicating the tripped condition stays lit only when the trip point is exceeded. Therefore, in the event of a spurious signal the trip unit light will only be illuminated momentarily. For this reason it was not possible to verify the specific switch that caused the isolation.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
Limerick Generating Station Unit 1		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

December 26, 1984

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with the automatic isolation of the RHR shutdown cooling inboard suction valve. This event occurred prior to initial criticality.

Reference:	Docket No. 50-352
Report Number:	84-019
Revision Number:	00
Event Date:	November 26, 1984
Report Date:	December 26, 1984
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10CFR50.73 (a) (2) (IV).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

IF22
1/1

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