

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 2 6 0				PAGE (3) 1 OF 0 2											
TITLE (4) Intergranular Stress Corrosion Cracking Indications on Unit 2																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
1	1	2	9	8	4	8	4	0	0	8	0	0	1	2	2	1	8	4	0	5	0	0	0	0	0
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																							
POWER LEVEL (10) 0 10 10		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)											
		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)											
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				X OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)				Part 21											
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME Alan W. Gordon										TELEPHONE NUMBER AREA CODE 2 0 5 7 2 9 - 2 5 3 7															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC															
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO													

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During inservice inspections, indications of intergranular stress corrosion cracking were found in the recirculation piping, reactor water cleanup piping, and jet pump instrument nozzles. The affected welds will be repaired prior to unit startup.

8501040057 841221
PDR ADOCK 05000260
S PDR

IE22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Browns Ferry - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 0 8 4 -	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			0 0 8 -	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal operation, unit 1 was operating at 99 percent, unit 2 was in a refueling outage, and unit 3 was in a startup mode. Only unit 2 was affected by this event.

In performing the inservice inspection in the unit 2 drywell, ultrasonic indications of intergranular stress corrosion cracking were found at four weld locations in the recirculation piping. These cracks ranged from .50 to 4 inches in length and from 10 to 26 percent through wall. One indication was found in the reactor water cleanup piping. This crack was .75 inches long and approximately 50 percent through wall.

All required inservice testing has been performed on unit 2 with seven weld areas yielding linear indications. Two of these areas were reported in LER 296/84006. The other five are reported above. Of the seven indication areas found, the limiting areas were located in the recirculation piping. If the recirculation piping indications had propagated to full length and 100 percent through wall, then the ensuing line break has been analyzed per FSAR 14.6.3.

The affected welds will be repaired prior to unit 2 restart. This is Part 21 reportable. No further corrective action is planned.

Responsible Plant Section - N/A

Previous Similar Events - BFRO-50-296/79-19, -260/82-40, 259/83023, -296/84006

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

December 21, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

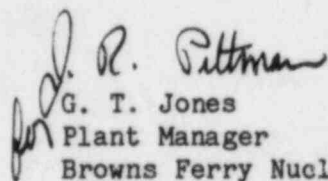
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/84008

The enclosed report provides additional details concerning intergranular stress corrosion cracking indications on unit 2. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(ii) and has been determined to be 10 CFR 21 reportable.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Resident Inspector, BFN

IE22
1/1