

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
LaSalle County Station Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 7 3

PAGE (3)

1 OF 0 3

TITLE (4)
Containment B & C Tests Exceed Tech Spec LimitEVENT DATE (5)
MONTH DAY YEAR
1 0 0 4 8 4
LER NUMBER (6)
YEAR SEQUENTIAL NUMBER REVISION NUMBER
8 4 - 0 6 4 - 0 1 1
REPORT DATE (7)
MONTH DAY YEAR
2 1 0 8 4
OTHER FACILITIES INVOLVED (8)
FACILITY NAMES
DOCKET NUMBER(S)
0 5 0 0 0
0 5 0 0 0THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)
OPERATING MODE (8)
POWER LEVEL (10)
0 0 0
20.402(b)
20.406(a)(1)(i)
20.406(a)(1)(ii)
20.406(a)(1)(iii)
20.406(a)(1)(iv)
20.406(a)(1)(v)
20.406(e)
30.36(e)(1)
30.36(e)(2)
30.73(a)(2)(i)
30.73(a)(2)(ii)
30.73(a)(2)(iii)
30.73(a)(2)(iv)
30.73(a)(2)(v)
30.73(a)(2)(vi)
30.73(a)(2)(vii)(A)
30.73(a)(2)(vii)(B)
30.73(a)(2)(x)
73.71(b)
73.71(e)
OTHER (Specify in Abstract below and in Text, NRC Form 265A)

LICENSEE CONTACT FOR THIS LER (12)

NAME
Robert D. Koenig, extension 575
TELEPHONE NUMBER
AREA CODE
8 1 5 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	S J	I S V	A 3 9 1	Y	X	S J	I S V	A 3 9 1	Y
X	B O	I S V	A 3 9 1	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On October 4, at 0500 hours, while performing LTS-100-9 (Feedwater and RWCU Return Valve Leak Rate Test Preparation) on Unit 1, it was discovered that valve 1B21-F010A (Feedwater Inboard Check Valve) was not fully seated. When a Type C LLRT was performed, it was found that the containment leakage limit for Type B & C leakage had been exceeded (0.6 la). The leakage rate was above 2100 SCFH; the limit is 231.4 SCFH.

The hinge pin bushing on one side of the check valve disc had moved out of the disc. This prevented the valve from fully closing. The bushing was subsequently replaced and pressed into the disc (0.001 inch, press fit), to prevent any bushing "moving-out".

Valves 1B21-F032B and 1E51-F064 were also found to exceed the allowable Technical Specification limits. These valves were repaired prior to the unit's restart.

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PDR ADCCK 05000373
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
LaSalle County Station Unit 1	0500037384	—	064	—01	02	OF 03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

I. EVENT DESCRIPTION

On October 4, 1984, at 0500 hours, while performing LTS-100-9 ("Feedwater (SJ) and RWCU (CE) Return Valve Leak Rate Test Preparation") on Unit 1, it was discovered that valve 1B21-F010A (Feedwater Inboard Isolation Check Valve) was not fully seated. This valve is a primary containment (NH) isolation valve and is required to close in the event of a loss of coolant accident (LOCA). When a type "C" local leakage rate test was performed, a leakage rate of greater than 2100 SCFH (Standard Cubic Feet per Hour) was found. This leakage rate placed the total Type B and C leakage for the Unit 1 containment above the limit of 0.6 La, 231.4 SCFH (Technical Specification 3.6.1.2.b).

Since this valve has failed, other valves have had the magnitude of leakage which would have placed the containment Type B & C leakage rate above 0.6 La. Valve 1B21-F032B, outboard feedwater check valve, was found to have a leakage rate of approximately 150 SCFH, and valve 1E51-F064, RCIC (BN) Steam Supply to RHR (BO) Steam Condensing, was found to have a leakage rate of approximately 150 SCFH.

II. CAUSE

The cause of 1B21-F010A not fully closing was the hinge pin bushing becoming wedged between the disc anti-rotation weld and the hinge pin shoulder. By design, the hinge pin bushing is press fit into the disc bore. The anti-rotation weld is applied to the disc to prevent the hinge pin bushing from rotating relative to the disc during power operations. It appears the hinge pin bushing to disc bore press fit was not maintained such that the bushing was able to move out of the disc bore during power operations. Once sufficiently out of the disc bore, the bushing was free to rotate relative to the disc. The hinge pin bushing shoulder subsequently became wedged between the anti-rotation weld and the hinge pin shoulder. This wedge prevented the valve from fully closing.

The cause for the 1B21-F032B not fully closing was a misalignment problem between the valve disc and body. This prevented the valve disc from fully seating.

The cause for the 1E51-F064 not completely sealing, was due to damage to the valve disc and seat. Steam cuts prevented the valve from sealing.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The feedwater lines have redundant isolation valves. In the event of a LOCA the feedwater line would have also been isolated by feedwater isolation valves 1B21-F065A/B and 1G33-F040. These isolation valves would limit the containment leakage from feedwater to well below Technical Specification limits.

The RCIC line has redundant isolation valves. Also, valves 1E51-F076 and 1E51-F063 would have isolated the RCIC penetration in the event of a LOCA.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
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LaSalle County Station Unit 1	0500037384	84	064	01	03	OF	03

TEXT (If more space is required, use additional NRC Form 386A's) (17)

IV. CORRECTIVE ACTIONS

The bushings in the 1B21-F010A feedwater check valve were replaced. The new bushings were pressed into the disc body (0.001 inch, press fit) to prevent any moving out of the bushing. A Local Leak Rate Test has been performed on the 1B21-F010A since repairs, and found to be well below limits (4.42 ± 0.05 SCFH). The 1B21-F010B was also tested during this outage as satisfactory. Even though it is felt that this is an isolated case, the Unit 2 valves, 2B21-F010A and B, will be leak rate tested at the next planned outage of sufficient duration. (AIR 1-84-247)

The alignment problem of the 1B21-F032B check valve was corrected by reboring the disc of the valve and installing new disc bushings. These repairs correct the valve alignment problem, which enabled the check valve to seat.

The disc and seat problem of the 1E51-F064 valve was corrected by relapping the disc and seat of the valve. These repairs corrected the damage (due to steam) done to the valve, which enabled the disc to seat fully.

Both valves were local leakage rate tested and found acceptable (leakages below 0.6 La).

V. PREVIOUS OCCURRENCES

Several failures of the feedwater check valves (failure to pass Type "C" tests) have occurred on Unit 1. (LER's 373/83-146/03X-1 and 373/84-012-00). No other failure of RCIC steam valves has occurred to date.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

R. D. Koenig, 815/357-6761, extension 575.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

December 10, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-064-01, Docket #050-373 is being submitted to your office to supercede previously submitted Reportable Occurrence Report 84-064-00.

R. D. Bishop
for G. J. Diederich
Station Superintendent

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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