

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
LaSalle County Station Unit 2DOCKET NUMBER (2)
0 5 0 0 0 3 7 4PAGE (3)
1 OF 0 3TITLE (4)
Reactor Water Cleanup Differential Flow Isolations

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
1	1	8	4	8	4	0	7	9	0	0	1	2	8	8	4	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																					
POWER LEVEL (10)	0 0 0	20.402(b)	20.408(a)(1)(i)	20.408(a)(1)(ii)	20.408(a)(1)(iii)	20.408(a)(1)(iv)	20.408(a)(1)(v)	20.408(e)	90.38(a)(1)	90.38(a)(2)	90.73(a)(2)(i)	90.73(a)(2)(ii)	90.73(a)(2)(iii)	90.73(a)(2)(iv)	90.73(a)(2)(v)	90.73(a)(2)(vi)	90.73(a)(2)(vii)(A)	90.73(a)(2)(vii)(B)	90.73(a)(2)(viii)	90.73(a)(2)(ix)	73.71(b)	73.71(e)	OTHER (Specify in Abstract below and in Text NRC Form 308A)

LICENSEE CONTACT FOR THIS LER (12)
NAME
John B. Reis, Jr., extension 463TELEPHONE NUMBER
AREA CODE
8 1 5 3 5 7 1 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs
X	J M	4 9 9 9	2 9 9 9	N					

SUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEARABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)
On 11/21/84 at 0158 hours, at less than 1% power in the Startup/Hot Standby Mode, the Unit 2 Reactor Water Cleanup System (CE, RT) isolated on High Differential Flow. At the time of the isolation, a filter demineralizer bed was being placed on line and the RT system was blowing down to the condenser. The RT system was checked for leaks and none were found. 12 minutes after the trip, the isolation signal was reset and the pumps were restarted with the filter demins in bypass. 88 minutes after the trip, 2 filter demins were satisfactorily placed in service. Combined with the additional system perturbation created by placing a filter demin. bed on line, such isolations are expected during Reactor Startup and/or Shut-down; the Reactor is at less than rated conditions, resulting in the Reactor Water Cleanup system operating at temperature, pressure and density conditions different than those specified in the differential flow instrument loop calibrations. Safe plant conditions were maintained at all times.IE 22
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	DIVISION NUMBER			
LaSalle County Station Unit 2	05000374	84	079	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 305A's) (17)

I. EVENT DESCRIPTION

On 11/21/84 at 0158 hours, at less than 1% power in Mode 2, Startup/Hot Standby, the Unit 2 Reactor Water Cleanup System (RT) isolated (JM) on High Differential Flow. At the time of the isolation, a filter demineralizer bed was being placed on line and the RT system was blowing down to the condenser. Also at this time, the Unit 2 Inboard Main Steam Isolation Valves (MSIV's, SB) were closed (since 0140 hours the same day) and the vessel was cooling down, awaiting repair of the Turbine Number 5 Bypass valve.

The RT system was checked for leaks and none were found. At 0210 hours on the same day the isolation was reset and the "A" and "B" RT pumps were restarted with the filter demins in bypass. At 0326 the same day the "A" and "C" filter demins were placed in service.

II. CAUSE

The Reactor Water Cleanup system normally operates with an indicated differential flow of 40 gpm with system isolation at 70 gpm. The Reactor Water Cleanup Flow Loops, which input to the Reactor Water Cleanup differential flow isolation logic, are calibrated for the Reactor operating at rated conditions. Combined with the additional system perturbation created by placing a filter demin bed on line, isolations can be expected during Reactor Startup and/or Shutdown; the Reactor is at less than rated conditions, resulting in the Reactor Water Cleanup system operating at temperature, pressure, and density conditions different than those specified in the differential flow instrument loop calibrations.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The isolation occurred in accordance with system design and Tech Spec Table 3.3.2-1, Trip Function 3.a. Safe plant conditions were maintained at all times. With the Reactor Water Cleanup system isolated, plant operations may continue (in either the Run, Startup/Hot Standby or Hot Shutdown Modes) as long as chemistry limitations are not exceeded. Prior to resetting the isolations and restarting the pumps, the Reactor Water Cleanup system was checked for leaks, with none being found.

As previously noted, less than 1 1/2 hours after the High Differential Flow Isolation signal was received, the Unit 2 Reactor Water Cleanup system was back in operation with 2 filter demins in service.

IV. CORRECTIVE ACTIONS

1. An investigation of the Reactor Water Cleanup areas revealed no actual leaks were present.
2. The system was promptly restarted with no difficulties.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		84	07	9	03	OF	03

TEXT (If more space is required, use additional NRC Form 385A's) (17)

IV. CORRECTIVE ACTIONS (CONTINUED)

3. AIR 01-84-67106 is outstanding to review the design temperature used to calibrate the flow instrumentation.
4. General Electric, the system vendor, is investigating the design basis for the isolation setpoint. (AIR 01-84-67137)

V. PREVIOUS OCCURRENCES

Previous events of this type occurred on Unit 1 and Unit 2 as described in the following LER's:

373/84-030	374/84-029
373/84-033	374-84-041
373/84-040	374-84-044
373/84-055	374/84-054
	374/84-057
	374/84-064
	374/84-073

VI. NAME AND TELEPHONE NUMBER OF PREPARER

John B. Reis, 815/357-6761, extension 463.



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LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

December 18, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-079-00, Docket #050-374 is being submitted to your office in accordance with 10CFR 50.73.

R. J. Diederich
for G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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