

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

LaSalle County Station Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 7 3 1 OF 0 3

PAGE (3)

TITLE (4)

Spurious Reactor Water Cleanup Differential Flow Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)													
1	1	2	4	8	4	8	4	0	8	2	0	1	2	1	9	8	4	0	5	0	0	0	0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																				
2			20.402(b)			20.408(e)			X			90.73(a)(2)(iv)			73.71(b)								
POWER LEVEL (10)			0 0 1			20.408(a)(1)(i)			90.36(a)(1)			90.73(a)(2)(v)			73.71(c)								
			20.408(a)(1)(ii)			90.36(a)(2)			90.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 388A)											
			20.408(a)(1)(iii)			90.73(a)(2)(i)			90.73(a)(2)(vii)(A)														
			20.408(a)(1)(iv)			90.73(a)(2)(ii)			90.73(a)(2)(vii)(B)														
			20.408(a)(1)(v)			90.73(a)(2)(iii)			90.73(a)(2)(viii)														

LICENSEE CONTACT FOR THIS LER (12)

NAME

John B. Reis, Jr., extension 463

TELEPHONE NUMBER

AREA CODE

8 1 5 3 5 7 1 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	JIM	Z19919	Z191219	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 24, 1984, at 1044 hours with Unit 1 at 1% power and in the Startup Mode, the Unit 1 Reactor Water Cleanup system isolated on High Differential Flow. There were no flowpath changes or equipment rotations in progress at the time of the isolation. The Reactor startup accounted for the isolation due to the water temperature, pressure, and density differences between actual startup operations and instrument calibrations (for normal operating conditions). Safe plant conditions were maintained at all times.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
LaSalle County Station Unit 1	05000373	84	082	00	02	OF	03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

I. EVENT DESCRIPTION

On 11/24/84 at 1044 hours the Unit 1 Reactor Water Cleanup system (CE, RWCU) isolated on High Differential Flow (JM). At the time of the occurrence, the unit was starting up (Mode No. 2), with Reactor power at approximately 1%, Reactor pressure at approximately 10 psig, and Reactor temperature at approximately 260°F. Also at this time the RWCU system was blowing down to the main condenser at the rate of approximately 100 gpm in order to maintain proper vessel level. No abnormal conditions associated with the RWCU system flow were noted. Upon actuation, the RWCU suction isolation valves 1G33-F001 and 1G33-F004 closed as required. Following isolation, the RWCU system was inspected for leakage, with none being found. Upon confirmation of satisfactory system status, the High Differential Flow Isolation signal was reset. At 1055 hours, the 1A RWCU pump was restarted with the filter demins in bypass. At 1100 hours the Reactor water was sampled by Rad Chem and found to be satisfactory, with a conductivity of .89 micromhos per cm and a pH of 5.8.

At 1210 the NRC was notified of the aforementioned isolation, and at 1330 the same day the 1C RWCU filter demin bed was satisfactorily placed on line.

II. CAUSE

The Reactor Water Cleanup system normally operates with an indicated differential flow of 40 gpm, with system isolation at 70 gpm. The Reactor Water Cleanup flow loops, which input to the Reactor Water Cleanup differential flow isolation logic, are calibrated for the Reactor operating at rated conditions. Isolations can be expected during Reactor Startup and/or Shutdown; the Reactor is at less than rated conditions, resulting in the Reactor Water Cleanup system operating at temperature, pressure, and density conditions different than those specified in the differential flow instrument loop calibrations.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The isolation occurred in accordance with system design and Technical Specification Table 3.3.2-1, Trip Function 3.a. Safe plant conditions were maintained at all times. With the Reactor Water Cleanup system isolated, plant operations may continue (in either the Run, Startup/Hot Standby or Hot Shutdown Modes) as long as chemistry specifications are not exceeded. Prior to resetting the isolations and restarting a pump, the Reactor Water Cleanup system was checked for leaks, with none being found.

As previously noted, less than 15 minutes after the High Differential Flow Isolation signal was received, the Unit 1 Reactor Water Cleanup system was back in operation.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

IV. CORRECTIVE ACTIONS

1. An investigation of the Reactor Water Cleanup areas revealed no actual leaks were present.
2. The system was promptly restarted with no difficulties.
3. AIR 01-84-67106 is outstanding to review the design temperature used to calibrate the flow instrumentation.
4. General Electric, the system vendor, is investigating the design basis for the isolation setpoint. (AIR 01-84-67137)

V. PREVIOUS OCCURRENCES

Previous events of this type occurred on Unit 1 and Unit 2 as described in the following LER's:

373/84-030	374/84-029
373/84-033	374/84-041
373/84-040	374/84-044
373/84-055	374/84-054
	374/84-057
	374/84-064
	374/84-073
	374/84-079

VI. NAME AND TELEPHONE NUMBER OF PREPARER

John B. Reis, 815/357-6761, extension 463.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

December 19, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-002-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

R. D. Bishop
for G. J. Diederich
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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