

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 7										PAGE (3) 1 OF 0 2	
---------------------------------------	--	--	--	--	--	--	--	--	--	--------------------------------------	--	--	--	--	--	--	--	--	--	----------------------	--

TITLE (4) Surveillance Requirement Not Met											
---	--	--	--	--	--	--	--	--	--	--	--

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
1	1	2	1	8	4	8	4	0	7	1	0	0	1	2	2	1	8	4	Sequoyah, Unit 2	0 5 0 0 0 3 2 8
												0 5 0 0 0								

OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)							
	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)							
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)								
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)								
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)										TELEPHONE NUMBER	
NAME Glenn E. Duggin, Compliance Section Engineer										AREA CODE 6 1 1 5	8 7 1 0 1 - 1 6 1 1 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During a review of Surveillance Instruction (SI)-6, "Containment Building Ventilation Isolation," it was noted that the radiation monitors for containment ventilation isolation were not being adequately verified in accordance with the technical specifications. The SI was verifying containment ventilation isolation from only the gaseous channels of the upper and lower containment radiation monitors, and did not include the particulate channels during core alterations or movement of irradiated fuel (as described in LCO 4.9.9) only. This procedure deficiency was discovered after the unit had returned to a nonapplicable mode. The SI is being revised to include both channels. Both channels of the radiation monitors were operable during the time that SI-6 was required as verified.

IE 22
1118501020468 841221
PDR ADOCK 05000327
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Sequoyah, Unit 1	0 5 0 0 0 3 2 7 8 4 —	0 7 1	—	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On November 21, 1984, with unit 1 in mode 1 at 100 percent power and unit 2 in mode 5, it was noted by Operations personnel that SI-6 did not adequately address the surveillance requirements for LCO 4.9.9. This LCO requires that a containment ventilation isolation be verified on a high radiation test signal from each of the containment radiation monitoring instrumentation channels. However, SI-6 required only a verification from the gaseous channel of these monitors (RM-90-106 and -112).

A review of the history of the SI was made and it was concluded that the most probable cause for the procedure deficiency was due to a misinterpretation of the surveillance requirement. It appears that monitor RM-90-106 for the lower containment compartment was considered one channel and RM-90-112 for the upper containment compartment was considered another, since only the gaseous channels are needed in the accident analysis. The particulate and gaseous channels for RM-90-106 and -112 are channel checked (C), channel calibrated (CC), and channel functional tested (CFT) in Modes 1, 2, 3, and 4 in accordance with applicable technical specification surveillances. The C, CC, and CFT are required shiftly, once per eighteen months, and quarterly, respectively, during modes 1 through 4.

Upon event discovery, the unit was in a nonapplicable mode for this LCO (4.9.9); however, a revision was initiated to SI-6 to address containment ventilation verifications from both the particulate and gaseous channels.

The revision to SI-6 should be complete by January 15, 1985. There was no effect upon public health or safety, and no plant safety margins were exceeded. There have been no previous occurrences.

TENNESSEE VALLEY AUTHORITY
Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

December 21, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84071

The enclosed licensee event report provides details concerning the failure to meet surveillance requirements as required by Technical Specifications. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.1.

Very truly yours,

TENNESSEE VALLEY AUTHORITY


P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

IE22
1/1