

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) PALISADES NUCLEAR PLANT	DOCKET NUMBER (2) 0 5 0 0 0 2 5 5	PAGE (3) 1 OF 0 2
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TITLE (4)

Primary Coolant System Unidentified Leakage Greater Than Limit

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	1	1984	84	02	5	0	0		NA		0 5 0 0 0
									NA		0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)										
	20.402(b)			20.408(a)			80.73(a)(2)(iv)			73.71(b)	
	20.408(a)(1)(i)			80.36(a)(1)			80.73(a)(2)(v)			73.71(a)	
	20.408(a)(1)(ii)			80.36(a)(2)			80.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 306A)	
	20.408(a)(1)(iii)			80.73(a)(2)(i)			80.73(a)(2)(vii)(A)				
POWER LEVEL (10) 0 1 0 1 0	20.408(a)(1)(iv)			80.73(a)(2)(ii)			80.73(a)(2)(vii)(B)				
	20.408(a)(1)(v)			80.73(a)(2)(iii)			80.73(a)(2)(viii)				

LICENSEE CONTACT FOR THIS LER (12)

NAME David W. Rogers; Technical Engineer; Palisades	TELEPHONE NUMBER	
	AREA CODE 6 1 6	7 6 4 - 8 9 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	C	B R V	F	0 3 5	Y				

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete E: EXPECTED SUBMISSION DATE)	X NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 19, 1984, with the Plant in hot shutdown, a leak rate test indicated primary coolant system unidentified leakage to be in excess of Technical Specification limits. Investigation determined that the leakage was from a relief valve in the Chemical and Volume Control System. The valve was exercised, and a subsequent leak rate test indicated normal leak rates within Technical Specification limits.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On November 19, 1984, with the Plant in hot shutdown, a two hour duration primary coolant system [AB] leak rate test indicated that unidentified leakage was in excess of the Technical Specification 3.1.5(a) limit. Calculations showed that unidentified leakage was 4.22 gallons per minute. The Technical Specifications limit for unidentified leakage is one gallon per minute.

Subsequent investigation determined that relief valve RV-2082 [RV;CB] was the source of the leakage. The valve was manually lifted, and a follow-up four hour duration leak rate test indicated that RV-2082 had seated properly. Calculations showed that unidentified leakage was reduced to 0.796 gallons per minute. This leak rate is within Technical Specifications limits.

Relief valve RV-2082 provides overpressure protection for the primary coolant pump [P;AB] controlled bleed-off piping [CB]. Evaluation of relief valve RV-2082 revealed that a bent stem had caused the valve to bind in an open position. No cause has been determined for the bent stem. The stem problem is not considered to have generic implications. Relief valve RV-2082 will be repaired when maintenance preparations are completed.

Although the leak rate exceeded Technical Specifications limits, no threat to public health or safety resulted. The source was easily identified, and the leakage was minimal. No reduction in plant operating conditions was required.



**Consumers
Power
Company**

General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

December 19, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-025 - PRIMARY COOLANT SYSTEM
UNIDENTIFIED LEAKAGE GREATER THAN LIMIT

Attached please find Licensee Event Report 84-025 - Primary Coolant System
Unidentified Leakage Greater Than Limit which is reportable to the NRC per
10 CFR 50.73(a)(2)(i).

Brian D Johnson

Brian D Johnson
Staff Licensing Engineer

CC Administrator, Region III, USNRC
Director, Office of Nuclear Reactor Regulation
NRC Resident Inspector - Palisades

Attachment

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