

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Point Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 3 6					PAGE 18 1 OF 2									
TITLE (4) Manual Reactor Trip																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)											
1	1	2	8	8	4	8	4	—	0	1	2	—	0	0	1	2	1	8	8	4	0 5 0 0 0			
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																					
POWER LEVEL (10)			20.402(b)				20.406(a)				X				50.73(a)(2)(iv)				73.71(b)					
0 6 2			20.406(a)(1)(i)				50.36(a)(1)				50.73(a)(2)(v)				73.71(a)									
			20.406(a)(1)(ii)				50.36(a)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 306A)									
			20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)													
			20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)													
			20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(viii)													
LICENSEE CONTACT FOR THIS LER (12)																								
NAME Steve Brinkman										TELEPHONE NUMBER														
										AREA CODE 2 0 3 4 4 7 - 1 7 9 1														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC														
B	S	D	H	X	Y	0	3	0	Y															
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)			MONTH DAY YEAR											
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO			0 8 3 0 8 5											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The Unit was operating in Mode 1 at 62% reactor power and was proceeding with an orderly shutdown necessitated by a leak in the #5A Feedwater Heater. Due to a steady rise in the level in the #5A Feedwater Heater, the Reactor was tripped in order to protect the turbine from water intrusion.

Reactor Trip and Post Trip actions were carried out in accordance with plant procedures. All systems functioned normally with the exception of the main feedwater check valve to the #2 Steam Generator (2-FW-5B), which failed to seat. Due to the malfunction of 2-FW-5B, initiation of Auxiliary Feedwater flow was required to maintain level in the #2 Steam Generator.

Reactor Trip recovery procedures were followed with personnel responding as expected.

Prior to returning the Unit to full power operation, all Feedwater Heater tube leaks were plugged and check valve 2-FW-5B was inspected and tested and was deemed operable. Check valve 2-FW-5B will be thoroughly inspected during the upcoming refueling outage (2/85).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Millstone Point Unit 2	DOCKET NUMBER (2) 0500033684	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		84	012	00	02	OF 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The Unit had been operating at 100% reactor power when a tube leak was detected in the #5A Feedwater Heater. An orderly shutdown of the Unit had been underway for some time when a steady rise in the level in the #5A Feedwater Heater was noted. As a result, in order to protect the turbine from water intrusion, the reactor was tripped. At the time of the trip, the Unit was operating in Mode 1 at 62% reactor power.

Following the trip, standard post trip actions were carried out in accordance with Operations Procedure 2525. In addition, reactor trip recovery procedure 2526 was also followed. All personnel and systems responded as expected with one notable exception. When the B-Steam Generator Feed Pump (SGFP) was taken to minimum speed, the level in both Steam Generators (SG) started decreasing. The B-SGFP was secured due to low discharge pressure and Auxiliary Feedwater (AFW) flow was initiated. The level in the #1 SG increased while the level in the #2 SG continued to decrease. AFW flow to the #2 SG was increased with no effect on the level decrease in the #2 SG. Operations personnel noted steam escaping from the B-SGFP. It was theorized that the main feedwater check valve to the #2 SG (2-FW-5B) failed to seat, thus resulting in a back-flow of AFW to the B-SGFP. At this time, the Feed Regulating valves, the Feed Regulating Block valves, and the SGFP discharge valves were shut. Level in the #2 SG was brought on scale and returned to normal.

The Unit was placed in hot standby (Mode 3). The tubes in the # 5A Feedwater Heater were inspected and plugged as required. The malfunctioning check valve, 2-FW-5B, was inspected. It was determined that the valve stem had bound up slightly not allowing the valve to seat. Adjustments were made to the valve and its operability was evaluated via inservice test, T84-36. The valve was deemed operable prior to returning the Unit to 100% power operation.

Corrective Action:

Check Valve 2-FW-5B will be thoroughly inspected during the upcoming refueling outage.

Previous Occurance:

None.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D C. 20555

Reference: Facility Operating License No. DPR-65
Docket No. 50-336
Reportable Occurrence RO 50-336/84-012

Gentlemen:

This letter forwards the Licensee Event Report 84-012-0 required to be submitted within thirty (30) days pursuant to paragraph 50.73 (a) (2) (IV) "Reactor Trip."

Very truly,

NORTHEAST NUCLEAR ENERGY COMPANY

E. J. Mroczka
Station Superintendent
Millstone Nuclear Power Station

EJM/SB:ejl

Attachment: LER RO 50-336/84-012

cc: Dr. T. E. Murley, Region I

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