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Resident Manager

June 15, 1992
JAFF-92-0464

United States Nuclear Regulatory Commission
Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

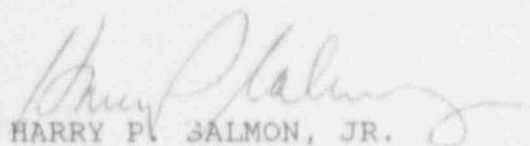
SUBJECT: DOCKET NO. 50-333
LICENSEE EVENT REPORT: 92-027-00 - Check Valve 14AOV-
13A Seat Leakage Exceeding
Technical Specification Limit

Dear Sir:

This report is submitted in accordance with 10 CFR 50.73(a)(2)(i).

Questions concerning this report may be addressed to
Mr. W. Verne Childs at (315) 349-6071.

Very truly yours,


HARRY P. SALMON, JR.

HPS:WVC:KA:lar

Enclosure

cc: USNRC, Region I
USNRC Resident Inspector
INPO Records Center

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20545, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

James A. FitzPatrick

Nuclear Power Plant

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

INTERIM REPORT

EIS Codes are in []

Description

During leak rate testing of the core spray [BM] A isolation testable check valve (14AOV-13A) on May 16, 1992, with the plant shutdown and in the cold condition for maintenance and refueling, seat leakage was determined to be in excess of the Technical Specification limit. Operating Technical Specification 4.7.A.2.d(1) requires the leakage rate for isolation valve 14AOV-13A to be less than 10 gpm when hydrostatically tested at 1,000 psig. During performance of the leak rate test, the check valve was unable to hold pressure using a hydro test pump producing 11.5 gpm at 1,100 psig. At the time of the test, with the drywell open for refueling, primary containment integrity was not required.

The valve involved is a 10-inch swing check manufactured by Atwood & Morrill. This type of valve, as used in our plant application, has exhibited problems during leak testing in the past when air was used as the test medium. This problem was corrected by changing to a hydrostatic test in 1978.

Cause

The apparent cause of the excessive seat leakage exhibited by isolation check valve 14AOV-13A was tight valve packing limiting the travel of the valve. The cause for the adverse valve packing condition will be evaluated and an update to this LER submitted to address the specific findings.

Analysis

The function of the core spray A isolation testable check valve is to restrict backflow in the A core spray line to prevent loss of reactor coolant outside the primary containment (drywell) in the event the core spray line breaks when the upstream containment isolation valves 14MOV-11A and -12A are open. Since the A core spray line containment isolation valves (14MOV-11A and -12A) were operable at all times, primary containment integrity and core spray low pressure piping overpressure protection was never compromised by the leaking check valve.

The excessive leakage by the isolation check valve constitutes a condition prohibited by Technical Specifications and is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B).

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) James A. FitzPatrick Nuclear Power Plant	DOCKET NUMBER (2) 0 5 0 0 0 3 3 3 9 1 2	LER NUMBER (6)			PAGE (3)		
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Corrective Action

1. The valve was repaired by replacing the swing arm packing in accordance with the packing installation procedure. While the valve was disassembled, the hinge pin bushings (which were slightly worn) were also replaced.
2. The valve will be hydrostatically retested for leakage to verify the adequacy of the repairs.
3. An update to this LER will be submitted to address any corrective actions determined by the outcome of the valve packing evaluation.

Additional Information

Failed Components: 14AOV-13A

Previous Similar Events: No previous events involving failure of this valve (or similar valves in other systems) when subjected to hydrostatic seat leak testing have occurred.

A supplemental report will be submitted to provide the additional information obtained from the evaluation of the valve packing condition causing this event.