Florida Power
CORPORATION
Crystal River Unit 3
Docket No. 50-302

March 20, 1996
3F0396-18

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Subject: Retraction of 10 CFR 50.72 Reportable Event No. 29998

Dear Sir:

This letter is to confirm the notification made via the Emergency Notification System (ENS) on February 29, 1996 retracting the notification made on February 19, 1996 for the subject event. The February 19, 1996 notification was made under 10CFR50.72(b)(1)(ii)(B) to report unacceptable as-found local leak rate testing results of Reactor Building (RB) Purge Inlet Containment Isolation Valves (AHV-1C and AHV-1D) discovered while Crystal River Unit 3 (CR-3) was in MODE 4 (HOT SHUTDOWN). The Improved Technical Specifications (ITS) Limiting Conditions for Operation (LCO) reported as not being met, were LCO 3.6.1 and LCO 3.6.3 for Containment and Containment Isolation Valve operability, respectively. Maintaining the containment OPERABLE requires compliance with the visual examinations and leakage rate test requirements of 10 CFR 50, Appendix J "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors."

Additional data determined that actual containment integrity was maintained. Troubleshooting performed on February 20, 1996 using the pneumatic soap-bubble test method revealed the leakage identified on February 19, 1996 was emanating from AHV-1D with no leakage from AHV-1C. AHV-1C is an electric motor operated valve inside the RB and AHV-1D is a pneumatic operated valve outside the RB. Since AHV-1C is maintained closed with power removed during MODES 1 through 4, there could be no single active failure that would have caused AHV-1C to fail open while AHV-1D was leaking. Therefore, using the guidance for minimum pathway leakage rate (the smallest leakage of two valves in a series) from ANSI N56.8 "Containment System Leakage Testing Requirements", it was demonstrated the above ITS limiting conditions were not applicable.

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If any additional information is required, please contact Mr. Kenneth R. Harrington at 352-563-4893.

Sincerely,

B J Hickle

B. J. Hickle, Director
Nuclear Plant Operations

TWC:ff

Attachment

xc: Regional Administrator, Region II
Project Manager, NRR
Senior Resident Inspector