

March 11, 1996



U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Subject: LaSalle County Nuclear Power Station Units 1 and 2
Submittal of Additional Information discussed during the March 11,
1996 Conference Call Regarding Modifying the Main Steamline Tunnel
Automatic Isolations
NRC Docket Nos. 50-373 and 50-374

References: (a) G. Benes letter to U. S. NRC, dated January 18, 1996, LaSalle
Submittal Regarding Main Steamline Tunnel Leak Detection
Isolations

(b) G. Benes letter to U. S. NRC, dated March 1, 1996, ComEd
Response to NRC Staff Request for Additional Information

Reference (a) provided LaSalle Station's proposal for revising the Technical
Specification requirements for the Main Steamline Tunnel Automatic Isolations.
Reference (b) provided ComEd's Response to the NRC staff request for additional
information. The purpose of this letter is to provide additional information that
was discussed during a conference call between the NRR Project Manager and
ComEd on March 11, 1996.

The engineering calculation, NED-I-EIC-0208, "Main Steam Tunnel Temperature
Isolation Setpoint Error Analysis", included with the Reference (b) submittal, was
prepared in accordance with the ComEd's setpoint error analysis methodology.
This methodology is documented in ComEd Technical Information Documents,
TID-E/I&C-10, "Analysis of Instrument channel Setpoint Error and Instrument
Loop Accuracy" and TID-E/I&C-20, "Basis for Analysis of Instrument Channel
Setpoint Error & Loop Accuracy". This methodology is consistent with ISA
Standard 67.04-1988, "Setpoints for Safety Related Instruments in Nuclear Power
Plants" which provides an acceptable method of analysis as described in RG1.105,
"Instrument Setpoint for Safety-Related Systems".

The ComEd setpoint methodology does not deviate from nor require ComEd to
take any exceptions to RG1.105. The ComEd methodology was previously
reviewed during the Systems Based Instrumentation and Control Inspection
(SBICI) performed at ComEd's Dresden Nuclear Power Stations,
August/September 1994. The inspection results, provided in report numbers 50-
237/94016(DRS) and 50-249/94016(DRS) "concluded that the methodology was

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technically sound, consistent with current industry practices, such as ISA 67.04-1988, and sufficiently comprehensive for their intended purpose."

The Technical Specification amendment request is for the Leak Detection system Group I isolation logic. This circuitry will remain on AC power supplies, as currently configured. The Reactor Building Ventilation system isolation logic is being changed to 125 VDC power to reduce the possibility of system isolations due to loss of power. This change to the power supplies is not being installed until L2R07/L1R08 (the first outage begins in September of 1996).

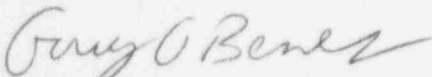
LaSalle has a reliable 125 volt DC distribution system. The Division 1 and Division 2 batteries each consist of 58 lead-calcium cells rated at 1200 amp-hours for an 8-hour discharge rate at 77° F. The Division 3 battery consists of 58 lead-calcium cells rated at 330 amp-hours for an 8-hour discharge rate at 77° F. These batteries are Gould batteries and consist of rectangular cells with plates that are vertically oriented within the cells. This type of cell construction has been used in the Nuclear industry for approximately 20 years, with a history of acceptable reliability.

The 125 volt batteries at LaSalle were replaced within the past 5 years due to aging, and for additional capacity to support the current plant configuration. The batteries have performed well since then, with one isolated cell failure (internal short). The failed cell was discovered during quarterly surveillance testing of the 125 volt DC batteries. This failure represents less than a 0.03% failure rate. The failed cell was replaced immediately.

The original Significant Hazards Consideration, that was included in the Reference (a) submittal, remains valid.

If there are any further questions, please contact this office.

Sincerely,



Gary G. Benes

Nuclear Licensing Administrator

cc: H. J. Miller, Regional Administrator - RIII
M. D. Lynch, Project Manager - NRR
P. G. Brochman, Senior Resident Inspector - LaSalle
Office of Nuclear Facility Safety - IDNS