



**Boston Edison**

Pilgrim Nuclear Power Station  
Rocky Hill Road  
Plymouth, Massachusetts 02360

**L. J. Olivier**

Vice President Nuclear Operations  
and Station Director

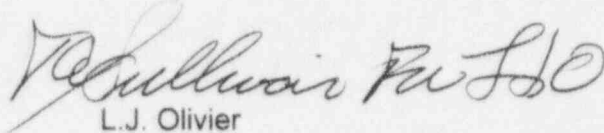
March 12, 1996  
BECO Ltr. #96-023

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Docket No. 50-293  
License No. DPR-35

**FEBRUARY 1996 MONTHLY REPORT**

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report, please contact me directly.

  
L.J. Olivier

RLC/dmc/9458

Attachment

cc: Mr. Thomas T. Martin  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Senior Resident Inspector

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# OPERATING DATA REPORT

DOCKET NO. 50-293  
 NAME: Pilgrim  
 DATE: March 12, 1996  
 COMPLETED BY: R. L. Cannon  
 TELEPHONE: (508) 830-8321  
 REPORT MONTH February, 1996

## OPERATING STATUS

## NOTES

- |  |   |
|--|---|
| <ol style="list-style-type: none"> <li>1. Unit Name</li> <li>2. Reporting Period</li> <li>3. Licensed Thermal Power (MWt)</li> <li>4. Nameplate Rating (Gross MWe)</li> <li>5. Design Electrical Rating (Net MWe)</li> <li>6. Maximum Dependable Capacity (Gross MWe)</li> <li>7. Maximum Dependable Capacity (Net MWe)</li> <li>8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:</li> </ol> | <p>Pilgrim I<br/>         February 1996<br/> <u>1998</u><br/> <u>678</u><br/> <u>655</u><br/> <u>696</u><br/> <u>670</u></p> <p>_____</p> <p style="text-align: center;">No Changes</p> |
|--|---|
- 
- |  |                                    |
|--|------------------------------------|
| <ol style="list-style-type: none"> <li>9. Power Level To Which Restricted, If Any (Net MWe):</li> <li>10. Reasons For Restrictions, If Any:</li> </ol> | <p><u>None</u><br/> <u>N/A</u></p> |
|--|------------------------------------|

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>696.0</u>	<u>1440.0</u>	<u>203592.0</u>
12. Hours Reactor Critical	<u>696.0</u>	<u>1440.0</u>	<u>127706.1</u>
13. Hours Reactor Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>696.0</u>	<u>1440.0</u>	<u>123263.9</u>
15. Hours Unit Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1390270.0</u>	<u>2772628.0</u>	<u>219074798.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>480620.0</u>	<u>956840.0</u>	<u>74234164.0</u>
18. Net Electrical Energy Generated(MWH)	<u>463698.0</u>	<u>922653.0</u>	<u>71366275.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>60.5</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>60.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>99.4</u>	<u>95.6</u>	<u>52.3</u>
22. Unit Capacity Factor (Using DER Net)	<u>101.7</u>	<u>97.8</u>	<u>53.5</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>11.9</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each) -	PLANNED MAINTENANCE OUTAGE, APRIL 19, 1996 FOR APPROXIMATELY SEVEN DAYS.		
25. If Shutdown at End of Report Period, Estimated Date of Startup -	UNIT OPERATING		

AVERAGE DAILY UNIT POWER LEVEL

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DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	663	17	666
2	665	18	667
3	664	19	666
4	665	20	666
5	665	21	666
6	665	22	657
7	665	23	666
8	668	24	666
9	668	25	666
10	668	26	666
11	668	27	666
12	667	28	667
13	667	29	666
14	667		
15	667		
16	667		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

## OPERATIONAL SUMMARY

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The unit started the reporting period at 100 percent core thermal power (CTP) where it was maintained until the end of the reporting period.

### SAFETY RELIEF VALVE CHALLENGES

#### MONTH OF FEBRUARY 1996

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

## REFUELING INFORMATION

DOCKET NO. 50-293  
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The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: February 1, 1997.
3. Scheduled date for restart following next refueling: March 14, 1997.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
7.
  - (a) There are 580 fuel assemblies in the core.
  - (b) There are 1765 fuel assemblies in the spent fuel pool.
8.
  - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
  - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

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SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER

No major safety related maintenance was completed during this reporting period.

# UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
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There were no unit shutdowns or significant power reductions during the reporting period.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)