



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

June 1, 1992

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Document Control Desk

Subject: Quad Cities Nuclear Power Station Units 1 and 2
Supplemental Response to Electrical Distribution
System Functional Inspection (EDSFI) Report
50-254/91011; 50-265/91007
NRC Docket Nos. 50-254 and 50-265

- References:
- (a) H.J. Miller letter dated June 24, 1991
transmitting NRC Inspection Report
50-254/91011; 50-265/91007
 - (b) T.J. Kovach letter to NRC dated July 24, 1991
transmitting CECO's response to IR 50-254/91011;
50-265/91007
 - (c) P.L. Piet letter to D. Butler dated April 27, 1992
transmitting Dresden ECCS Full Flow Functional Test
 - (d) H.J. Miller letter to Cordell Reed dated May 8, 1992
EDG Response acknowledgement

Reference (b) provided the Commonwealth Edison Company (CECo) response to the EDSFI report of Reference (a). As a result of a discussion between Mr. David Butler, NRC Region III, and CECO, the actions to be taken to address Unresolved Item 254/910011-03; 265/91007-03 were revised. Attachment A provides the revised CECO response and current status to the subject Unresolved Item.

If your staff has any questions or comments concerning this letter, please contact Jim Watson, Compliance Engineer, at (708) 515-7205.

Sincerely,

P. L. Banner for

T.J. Kovach
Nuclear Licensing Manager

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PDR ADDCK 05000254
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Attachments

cc: A.B. Davis, Regional Administrator- Region III
D. Butler, Region III
T. Taylor, Senior Resident Inspector
L. Olshan, Project Manager- NRR

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ATTACHMENT A
UNRESOLVED ITEM
254/91011-03; 265/91007-03

During the EDSFI inspection, the team identified a number of weaknesses with calculation No. 7318-33-19-3, dated October 5, 1990, in regard to the capability of the Diesel Generator (DG) to meet load sequencing and total load capacity requirements.

Also, it was unclear how a designated design reviewer could determine calculation acceptability since key acceptance criteria of the calculation and DG loading capability criteria were not stated in the body of the calculation. Pending further analysis by the licensee and subsequent NRC review, this item was considered unresolved.

CECo Response:

During the EDSFI inspection at Quad Cities Station, the NRC team members identified weaknesses with the Emergency Diesel Generator loading calculations. Originally, CECo committed to revise the Diesel Generator calculations by the end of the first quarter of 1992.

On February 19, 1992, Chris Grier (CECo Engineering) and Mr. David Butler (NRC-Region III) discussed improved alternative actions to resolve this issue. At that time, Mr. Butler acknowledged the revised actions which were:

1. Complete an evaluation of Dresden's Diesel Generators Automatic Load Sequencing for Units 2 and 3 (DG 2, 3, and 2/3). This evaluation shows the relationship between Emergency Diesel Generator Loading Sequence and the DBA LOCA Accident Scenario. Results of the evaluation shows the diesel generator response to the automatic load sequence adequately characterizes the DG response during an actual DBA. This report was sent to Mr. Butler in the letter of Reference (c) and acknowledged in the letter of Reference (d). CECo has reviewed this evaluation and verified that it is applicable to Quad Cities Station.
2. Prepare a project plan to determine the exact scope, schedule, and resource requirements to perform a EDG dynamic analysis for Dresden Station (also applicable to Quad Cities Station). The objective of this report is to present a generic methodology for performing dynamic DG loading analysis at CECo nuclear power plants.

The above mentioned reports have been completed. These reports resolve the NRC concerns on the EDG loading calculations and the ability of the EDG's to handle the required emergency electrical loads at Dresden and Quad Cities Stations.