

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346

UNIT Davis-Besse #1

DATE April 13, 1992

COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

MONTH March, 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>888</u>
2	<u>3</u>	18	<u>887</u>
3	<u>790</u>	19	<u>889</u>
4	<u>887</u>	20	<u>890</u>
5	<u>883</u>	21	<u>891</u>
6	<u>882</u>	22	<u>887</u>
7	<u>883</u>	23	<u>891</u>
8	<u>884</u>	24	<u>887</u>
9	<u>885</u>	25	<u>888</u>
10	<u>887</u>	26	<u>888</u>
11	<u>890</u>	27	<u>887</u>
12	<u>889</u>	28	<u>890</u>
13	<u>889</u>	29	<u>888</u>
14	<u>891</u>	30	<u>887</u>
15	<u>891</u>	31	<u>890</u>
16	<u>886</u>		

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

# OPERATING DATA REPORT

DOCKET NO. 50-346  
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## OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
2. Reporting Period: March, 1992
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 921
7. Maximum Dependable Capacity (Net MWe): 877
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr. to-Date	Cumulative
11. Hours In Reporting Period	744.0	2,184.0	119,833
12. Number Of Hours Reactor Was Critical	719.2	2,159.2	68,335.0
13. Reactor Reserve Shutdown Hours	24.8	24.8	5,532.0
14. Hours Generator On-Line	702.3	2,142.3	66,175.4
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,930,643	5,917,666	162,526,959
17. Gross Electrical Energy Generated (MWH)	648,250	1,989,784	53,883,161
18. Net Electrical Energy Generated (MWH)	615,000	1,892,058	50,684,546
19. Unit Service Factor	94.4	98.1	55.2
20. Unit Availability Factor	94.4	98.1	56.7
21. Unit Capacity Factor (Using MDC Net)	94.3	98.8	48.2
22. Unit Capacity Factor (Using DER Net)	91.2	95.6	46.7
23. Unit Forced Outage Rate	5.6	1.9	24.6
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346

UNIT NAME Davis-Besse #1

DATE April 13, 1992

COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

REPORT MONTH March, 1992

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	92-03-01	F	10.4	H	5	NA	NA	NA	Power reduction to approximately 6 percent to take the turbine-generator off line to allow containment entry for Containment Air Coolers cleaning and to perform containment walkdown and inspection for leaks.
2	92-03-01	F	31.3	H	3	92-002	SB	PCV	The reactor tripped by the Anticipatory Reactor Trip System (ARTS) following a turbine trip.  See Operational Summary for further details.

<sup>1</sup> F: Forced

S: Scheduled

<sup>2</sup> Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training &amp; License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

<sup>3</sup> Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Continuation from  
Previous Month

5-Load Reduction

9-Other (Explain)

<sup>4</sup> Exhibit G - Instructions for Preparation of Data

Entry Sheets for Licensee Event Report (LER)

File (FORMS-0161)

<sup>5</sup> Exhibit I - Same Source\*Report challenges to Power Operated Relief Valves  
(PORVs) and Pressurizer Code Safety Valves (PCSVs)

Operational Summary  
March, 1992

Reactor power reduction to approximately 6 percent continued. A manual power reduction to 6 percent was initiated to take the turbine-generator off line to allow containment entry for Containment Air Coolers (CACs) cleaning and inspection for the source of a small Reactor Coolant System (RCS) leak that was causing boron deposits on the Containment Air Coolers.

The turbine-generator was taken off line at 0326 hours on March 1, 1992.

At 1157 hours on March 1, 1992, and after the completion of Containment Air Coolers cleaning, reactor power was slowly increased and the turbine-generator was synchronized on line at 1348 hours on March 1, 1992.

At 1500 hours on March 1, 1992, while the unit was at approximately 40 percent of full power, a reactor trip occurred. The reactor tripped by the Anticipatory Reactor Trip System (ARTS) following a turbine trip. The turbine trip was due to actuation of the exhaust hood temperature trip circuit when Maintenance - I&C personnel were checking the condition of the vacuum pressure switches at a junction box local to the turbine. It has been determined that a meter test lead came into contact with the wrong terminal point, which then actuated the turbine exhaust hood high temperature trip for the main turbine.

The reactor was critical at 1530 hours on March 2, 1992, and the turbine-generator was synchronized on line at 2221 hours on March 2, 1992.

Reactor power was slowly increased to approximately 100 percent, which was achieved at 0625 hours on March 3, 1992, and maintained at this power level for the rest of the month.

REFUELING INFORMATION

Date: March 1992

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling outage? March 1993
3. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, and (c) the new fuel storage areas.

(a) 177 (b) 393 (c) 0

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735

5. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1994 - assuming ability to unload the entire core into the spent fuel pool is maintained