SOUTHERN CALIFORNIA EDISON
An EDISON INTERNATIONAL Company

R. W. Krieger
Vice President
Nuclear Generation

March 8, 1996

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Subject: Docket Nos. 50-361 and 50-362
30 Day Report
Licensee Event Report No. 96-002
San Onofre Nuclear Generating Station, Units 2 and 3

This submittal provides a written Licensee Event Report (LER) for a condition that could potentially cause the logarithmic power level indication to become decalibrated. Since this condition is applicable to Units 2 and 3, a single report for Unit 2 is being submitted in accordance with NUREG-1022. Neither the health nor the safety of plant personnel or the public was affected by this occurrence.

Sincerely,

Enclosure: LER No. 96-002

cc: L. J. Callan, Regional Administrator, NRC Region IV
J. E. Dyer, Director, Division of Reactor Projects, NRC Region IV
K. E. Perkins, Jr., Director, Walnut Creek Field Office, NRC Region IV
J. A. Sloan, NRC Senior Resident Inspector, San Onofre Units 2 & 3
M. B. Fields, NRC Project Manager, San Onofre Units 2 and 3
Institute of Nuclear Power Operations (INPO)

P. O. Box 128
San Clemente, CA 92674
714-368-6255
Fax 714-368-6183

IE22

LICENSEE EVENT REPORT (LER)

Facility Name (1)	Docket Number (2)	Page (3)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2	0 5 0 0 0 3 6 1	1 of 0 1

Title (4)

Potential Decalibration of Logarithmic Power Level Instrumentation

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
Month	Day	Year	Year	///	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0 2	0 9	9 6	9 6	---	0 0 2	--- 0 0	0 3	0 8	9 6	SONGS UNIT 3	0 5 0 0 0 3 6 2

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)					
POWER LEVEL (10)	0 9 9	___ 20.402(b)	___ 20.40(c)	___ 50.73(a)(2)(iv)	___ 73.71(b)		
//////////////////////////		___ 20.405(a)(1)(i)	___ 50.36(c)(i)	X 50.73(a)(2)(v)	___ 73.71(c)		
//////////////////////////		___ 20.405(a)(1)(ii)	___ 50.36(c)(2)	___ 50.73(a)(2)(vii)	X Other (Specify in Abstract below and in text) 10CFR21		
//////////////////////////		___ 20.405(a)(1)(iii)	___ 50.73(a)(2)(i)	___ 50.73(a)(2)(viii)(A)			
//////////////////////////		___ 20.405(a)(1)(iv)	___ 50.73(a)(2)(ii)	___ 50.73(a)(2)(viii)(B)			
		___ 20.405(a)(1)(v)	___ 50.73(a)(2)(iii)	___ 50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)

Name	TELEPHONE NUMBER AREA CODE
R. W. Krieger, Vice President, Nuclear Generation	7 1 4 3 6 8 - 6 2 5 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC.-TRUER	REPORTABLE TO NPRDS	///////	CAUSE	SYSTEM	COMPONENT	MANUFAC.-TRUER	REPORTABLE TO NPRDS	///////
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SUPPLEMENTAL REPORT EXPECTED (14)		Expected Submission Date (15)	Month	Day	Year
___ Yes (If yes, complete EXPECTED SUBMISSION DATE)	___ X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 2/9/96, ABB Combustion Engineering (CE) notified Edison that a potential non-conservatism in the calibration of logarithmic (log) power channels [IG] reported by the Waterford Station (Waterford LER 50-382/96-003) may exist at San Onofre Units 2 and 3. At 10:45 PST, Edison determined that the Waterford condition did apply to SONGS 2 and 3, and existed during previous plant startups. Edison provided a 10CFR50.72(b)(2)(iii)(D) report to the NRC at 1427 PST (Log No. 29965).

This condition resulted from calibrating the log power channels at 100% power (chosen to allow correlation to plant power from heat balance methods). While this method is accurate at full power, physics effects may cause the log power indication to vary significantly from actual plant power during low power operation (CEA position, temperature shadowing, boron changes, etc.). CE estimated these effects could cause the log power trip [JC] to be high by a factor of two but less than a factor of 10. Due to the passage of time, it is not known why these effects were not considered in the original CE calculation (circa 1980).

Edison will reduce the trip setpoint by a factor of 10 until the effects have been confirmed by analyses. If either unit is shutdown prior to adjusting the trip setpoint (trip required only during startup), the reactor trip breakers will remain open until the setpoints are adjusted. Edison is working with CE for a long term solution.

Edison believes the setpoint analysis will likely confirm the original trip setpoints were acceptable. There have been no instances in the past which would have required a high log power trip at either unit and the safety significance is believed to be minimal. If the analysis determines otherwise, Edison will revise this LER. Edison has not reported any similar events in the past three years.