



Commonwealth Edison
LaSalle County Nuclear Station
2601 N. 21st Rd.
Marseilles, Illinois 61341
Telephone 815/357-6761

March 19, 1992

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

Dear Sir:

Licensee Event Report #85-035-01, Docket #050-374 is being submitted to your office in accordance with 10CFR 50.73.

WRO
for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MWM/tsh

Enclosure

xc: Nuclear Licensing Administrator
NRC Resident Inspector
NRC Region III Administrator
INPO - Records Center
IDNS Resident Inspector

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LICENSEE EVENT REPORT (LER)

Form Rev 2-0

Facility Name (1) LaSalle County Station Unit 2 Docket Number (2) 0 | 5 | 0 | 0 | 0 | 3 | 7 | 4 Page (3) 1 of 0 | 3
 Title (4)

Actuation Of E SRV/SRV N Actuation

Event Date (5)			LER Number (6)			Report Date (7)			Other Facilities Involved (8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0 7 2 3 8 5	8 5	---	0 3 5	---	0 1	0 3 1 9 9 2				0 5 0 0 0 1 1

OPERATING MODE (9) 1

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

POWER LEVEL (10) 0 2 3	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)	73.71(b)
	20.405(a)(i)(v)	50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
	20.405(a)(i)(ii)	50.36(c)(2)		50.73(a)(2)(vii)	Other (Specify
	20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	in Abstract
	20.405(a)(1)(iv)	X 50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	below and in
	20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)	Text)

LICENSEE CONTACT FOR THIS LER (12)

Name: Michael W. McLain, Extension 2701

TELEPHONE NUMBER: AREA CODE 8 | 1 | 5 | 3 | 5 | 7 | - | 6 | 7 | 6 | 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
D	S	B	R	V	C	7	1	0	N

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15) Month | Day | Year

Yes (If yes, complete EXPECTED SUBMISSION DATE) X | NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 23, 1985, during the performance of LaSalle Special Test (LST) 85-103, Set Pressure Verification Device (SPVD) Operational Test, Safety Relief Valve (SRV-SB) 2B21-F013E was inadvertently opened twice and Safety Relief Valve 2B21-F013N was inadvertently opened once.

At the time of this event LaSalle Unit 2 was in Operating Condition 1 at 23% power. The cause of the inadvertent actuations was performance of the SPVD Operational Test at an excessive pressure (955 psig). The SPVD is designed to test the SRV setpoints at a system pressure of 300 psig below the nameplate set pressure of the SRV. This results in a test pressure range of 850-905 psig reactor pressure.

This event was of minimal significance because both SRV's reclosed as indicated by the tailpipe temperatures returning to their prelift values.

The SRV's were verified to be closed by observing tailpipe temperatures. The SPVD Operational Test was terminated to preclude any additional SRV actuations. Future testing of the Set Pressure Verification Device will only be performed at a reactor pressure between 850-905 psig.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION												Form Rev 2.0	
FACILITY NAME (1)			DOCKET NUMBER (2)				LER NUMBER (6)					Page (3)	
							Year	Sequential Number		Revision Number			
LaSalle County Station Unit 2			0 5 0 0 0 3 7 4				8 5	-	0 3 5	-	0 1	0 2 OF 0 3	
TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]													

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

A. CONDITION PRIOR TO EVENT

Unit(s): 2 Event Date: 07/23/85 Event Time: 1124 Hours

Reactor Mode(s): 1 Mode(s) Name: Run Power Level(s): 23%

B. DESCRIPTION OF EVENT

On July 23, 1985, during the performance of LaSalle Special Test (LST) 85-103, Set Pressure Verification Device (SPVD) Operational Test, Safety Relief Valve 2B21-F013E was inadvertently opened twice and Safety Relief Valve 2B21-F013N was inadvertently opened once.

C. APPARENT CAUSE OF EVENT

At the time of this event, LaSalle Unit 2 was in Operating Condition 1 at approximately 23% power. At the start of testing, Station personnel were not aware of any maximum pressure restrictions for SPVD testing, i.e., none are listed in the SPVD technical manual. The cause of these inadvertent Safety Relief Valve (SRV-SB) actuations was performance of the SPVD operational test at an excessive system pressure (955 psig). Following these two valve actuations, a Crosby Valve and Gage Company representative informed Station personnel that the SPVD was designed to test the set pressure of the SRV's at a system pressure of 300 psig below the nameplate set pressure of the SRV. This results in a pressure range of 850-905 psig, reactor pressure, in which this test can be performed. Due to the excessive pressure, the force on the under side of the valve disc is sufficient to cause the SRV to fully open when nitrogen is admitted to the SPVD bellows assembly, mounted on the top of the SRV, during the test. When the nitrogen pressure was vented from the bellows, the SRV's reclosed. These valves are 6 X R X 10 style HB-65-BP Dual Function Safety Relief Valves, manufactured by Crosby Valve and Gage Company.

D. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

This event was of minimal significance because the SRV's both reclosed as indicated by the tailpipe temperature returning to their prelift values.

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		Year	///	Sequential Number	///	Revision Number						
LaSalle County Station Unit 2	0 5 0 0 0 3 7 4	8 5	-	0 3 5	-	0 1	0 3	OF	0 3			
TEXT Energy Industry Identification System (EIIIS) codes are identified in the text as [XX]												

E. CORRECTIVE ACTIONS

The SRV's were verified to be closed by verifying the SRV tailpipe temperatures returned to the prelift value. The Set Pressure Verification Device Operational Test, LST-85-103, was terminated to preclude any additional SRV actuations. Future testing of the Set Pressure Verification Device, on the Unit 2 SRV's will only be performed at a reactor pressure between 850-905 psig. Action Item Record 374-200-85-00128 determined that a SPVD Operational Test was not needed, based on cost effectiveness of in-house setpoint testing. At the present time LaSalle County Station sends SRV's to Wyle Labs for setpoint testing and seat leakage determination and repair.

F. PREVIOUS EVENTS

None.

G. COMPONENT FAILURE DATA

None.