



Commonwealth Edison
LaSalle County Nuclear Station
2601 N. 21st. Rd.
Marseilles, Illinois 61341
Telephone 815/357-6761

March 10, 1992

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for February, 1992.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

xc: A. B. Davis, NRC, Region III
D. E. Hillis, NRC Resident Inspector LaSalle
J. L. Roman, IL Dept. of Nuclear Safety
B. Siegel, NRR Project Manager
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
D. R. Eggett, NED
P. D. Doverspike, GE Resident
T. J. Kovach, Manager of Nuclear Licensing
W. F. Naughton, Nuclear Fuel Services Manager
J. E. Lockwood, Regulatory Assurance Supervisor
J. W. Gieseke, Technical Staff Supervisor
M. M. Servoss, QA/NS Off Site Review
Station File

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

FEBRUARY 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815) 357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1130 Mwe.
3	0030	Reduced power level to 1000 Mwe due to system load.
	0900	Increased power level to 1130 Mwe.
4	0100	Reduced power level to 850 Mwe due to system load, performed Turbine Control Valve cycling.
	0700	Increased power level to 1125 Mwe.
5	0100	Reduced power level to 865 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
10	0100	Reduced power level to 900 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
	2330	Reduced power level to 950 Mwe to perform Turbine Control Valve testing and Reactor Protection monthly surveillances.
11	0900	Increased power level to 1130 Mwe.
17	0200	Reduced power level to 1030 Mwe due to system load.
	0900	Increased power level to 1130 Mwe.
18	0130	Reduced power level to 950 Mwe to perform monthly surveillance on the # 4 Turbine Control Valve.
	1000	Increased power level to 1130 Mwe.
20	0300	Reduced power level to 880 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
22	1500	Reduced power level to 850 Mwe due to system load.
23	1800	Increased power level to 1130 Mwe.
25	0100	Reduced power level to 1000 Mwe due to system load, performed Reactor Protection Monthly surveillances.
	1000	Increased power level to 1130 Mwe.
29	2400	Reactor critical, Generator on-line at 1130 Mwe.

D. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Amend Division 3 battery specifications for float voltage and specific gravity.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
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(None.)

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L02793	'B' Control Room HVAC Return Fan Isolation Damper OVC15YB	Differential pressure switch OPDS-VC051	Damper cycled repeatedly during surveillance testing.	Recalibrated pressure switch.
L12959	Suppression Pool Water Bulk Temperature Monitor IUY-CN037	RTD input module.	Inaccurate temperature readings.	Replaced RTD input module.
L13469	Suppression Pool Clean 'A' phase moveable arcing Up and Transfer Pump 7SF01P8	contact cracked.	None.	Replaced contact.
L13745	Control Room HVAC Ammonia Detector GXY-VC125A	Broken cassette carrier.	Detector inoperable.	Replaced cassette carrier.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE March 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 557-6761

OPERATING STATUS

1. REPORTING PERIOD: February 1992
GROSS HOURS IN REPORTING PERIOD: 696

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
MAX DEPENDABLE CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	696.0	1,440.0	48,796.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	696.0	1,440.0	47,818.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWht)	2,263,836	4,673,011	139,829,990
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	774,006	1,599,080	46,688,852
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	753,594	1,554,424	44,749,569
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	68.2
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	70.5
14. UNIT SERVICE FACTOR (%)	100.0	100.0	66.8
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	66.8
16. UNIT CAPACITY FACTOR (USING MDC) (%)	104.5	104.2	60.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	100.4	100.1	58.0
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	7.4

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE March 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: March 1992

DAY	POWER	DAY	POWER
-----	-----	-----	-----
1	1,108	17	1,099
2	1,106	18	1,064
3	1,071	19	1,105
4	1,043	20	1,060
5	1,044	21	1,103
6	1,105	22	989
7	1,106	23	998
8	1,103	24	1,094
9	1,105	25	1,039
10	1,044	26	1,100
11	1,069	27	1,101
12	1,108	28	1,101
13	1,109	29	1,102
14	1,110	30	
15	1,109	31	
16	1,106		

TABLE A

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit: 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
(None.)						

SUMMARY OF OPERATION:

The Unit remained on-line at high power throughout the month. Several minor power reductions were required due to system loading.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None)

2. ECCS System Outages
(See Table 5)

3. Changes to the Off-Site Dose Calculation Manual
(None.)

4. Major changes to Radioactive Waste Treatment Systems.
(None.)

5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
0079	ODG01K	Administrative control during Unit 2 output breaker testing.
(U-1)		
0067	1E12-C300A 1E12-C300B	Pump lubrication.
0120	1DG01P 1DO01P	Pump lubrication.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

FEBRUARY 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Ciaikowski, telephone number (815)337-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u> _____
1	0000	Reactor subcritical, Generator off-line, Refuel outage (L2R04) in progress.
29	2400	Reactor subcritical, Generator off line, Refuel outage (L2R04) in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Amend Division 3 battery specifications for float voltage and specific gravity.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-002-00	02/26/92	A spurious Division I ECCS initiation was received when a false low level signal was sensed due to a pressure spike on the reference leg.
92-003-00	02/28/92	During the 0 Diesel Generator response time test, the Unit 2 output breaker (ACB 2413) failed to close during a simulated under-voltage on bus 241Y.

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L00514	Diesel Generator Cylinder Head Temperature	Cylinder head thermocouple.	Inaccurate cylinder head temperature indications.	Replaced thermocouple.
L11883	Main Steam Line Low Pressure Switch 2B21-N015B	Pressure switch vent valve leaking.	None.	Replaced valve stem.
L12667	Standby Gas Treatment System Moisture Separator Differential Pressure	Differential pressure indicator.	Inaccurate pressure indication.	Replaced indicator.
L13174	High Pressure Core Spray Diesel Generator Lube Oil Low Pressure	Pressure switch 2E22-N513.	Erratic switch actuation.	Replaced pressure switch.
L13269	Diesel Generator Left Bank Air Supply	Pressure gauge 2PI-DG098A.	Improper pressure indication.	Replaced pressure gauge.
L13293	High Pressure Core Spray Diesel Generator Cooling Water Hot Engine Temperature	Temperature switch 2E22-N516.	None.	Replaced temperature switch.
L13396	High Pressure Core Spray Diesel Generator	HFA relay.	Intermittent grounds.	Replaced HFA relay.

C. TABLE 1 (Unit 2) —CONTINUED

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L13446	Hydraulic Control Unit Accumulator 58-39	Pressure switch 2C11-N138PS	Pressure switch failed during surveillance testing.	Replaced pressure switch.
L13456	Low Pressure Core Spray Minimum Flow	Pressure switch.	Switch was found out of tolerance.	Replaced switch.
L13653	Hydraulic Control Unit 38-47	Nitrogen block valve leaking.	None.	Replaced va've cartridge.
L13670	Suppression Chamber Spray Stop Valve 2E12-F027A	Packing leak.	None.	Performed packing adjust.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE March 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:
GROSS HOURS IN REPORTING PERIOD:

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
MAX DEPENDABLE CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	84.0	45,019.8
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.0
7. GENERATOR ON-LINE TIME (HOURS)	0.0	73.9	44,278.5
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWht)	0.0	180,986	132,423,385
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	0.0	64,274	43,954,024
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	-7,682	46,479	42,177,983
12. REACTOR SERVICE FACTOR (%)	0.0	5.8	69.7
13. REACTOR AVAILABILITY FACTOR (%)	0.0	5.8	72.4
14. UNIT SERVICE FACTOR (%)	0.0	5.1	68.6
15. UNIT AVAILABILITY FACTOR (%)	0.0	5.1	68.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	3.1	63.0
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	-1.0	3.0	60.6
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	13.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
REFUELING (L2R04) 01/04/92 11 WEEKS

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
04/02/92

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE March 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: March 1992

DAY	POWER	DAY	POWER
-----	-----	-----	-----
1	-11	17	-11
2	-11	18	-11
3	-11	19	-11
4	-11	20	-11
5	-11	21	-11
6	-11	22	-11
7	-11	23	-11
8	-11	24	-11
9	-11	25	-11
10	-11	26	-11
11	-11	27	-11
12	-11	28	-11
13	-11	29	-11
14	-11	30	
15	-11	31	
16	-11		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL DATE NUMBER	(YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
01	920104	S	696.0	C	1	Refueling outage L2R04.

SUMMARY OF OPERATION:

The unit entered a scheduled refueling outage 01/04/92.

UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None.)

2. ECCS System Outages
(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.
(None.)

4. Major changes to Radioactive Waste Treatment Systems.
(None.)

5. Indications of Failed Fuel Elements.
(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1115 1188	2DG08CB	Repair relief valve.
1214	2DG08CB	Repair compressor.