

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
PALISADES NUCLEAR PLANTDOCKET NUMBER (2)
0 5 0 0 0 2 5 5 1 OF 0 2TITLE (4)
Reactor Trip Set Less Than Technical Specification LimitEVENT DATE (5)
MONTH DAY YEAR
11 09 84
LER NUMBER (6)
YEAR SEQUENTIAL NUMBER REVISION NUMBER
84 - 023 - 010
REPORT DATE (7)
MONTH DAY YEAR
11 09 84
OTHER FACILITIES INVOLVED (8)
FACILITY NAMES
NA
DOCKET NUMBER(S)
0 5 0 0 0THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)
OPERATING MODE (9) N
POWER LEVEL (10) 0100
20.402(b) 20.406(c) 80.73(a)(2)(iv) 73.71(b)
20.406(a)(1)(i) 80.38(a)(1) X 80.73(a)(2)(v) 73.71(a)
20.406(a)(1)(ii) 80.38(a)(2) X 80.73(a)(2)(vi) OTHER (Specify in Abstract below and in Text, NRC Form 205A)
20.406(a)(1)(iii) X 80.73(a)(2)(vii)(A)
20.406(a)(1)(iv) X 80.73(a)(2)(vii)(B)
20.406(a)(1)(v) 80.73(a)(2)(viii)LICENSEE CONTACT FOR THIS LER (12)
NAME
David W. Rogers; Technical Engineer; Palisades
TELEPHONE NUMBER
AREA CODE 616 764 - 8913COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRC
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NRCSUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 9, 1984, with the plant in cold shutdown, the Reactor Protection System low primary coolant flow trip was determined to be improperly set. A low flow trip calibration was not scheduled and subsequently not performed when required, resulting in incorrect trip values in a Reactor Protection System surveillance procedure. As a result, a Technical Specification safety system setpoint limit was exceeded during previous plant operation.

The calibration procedure was subsequently performed and the correct low flow trip values provided for the surveillance procedure. The calibration procedure requirements will be included in the surveillance procedure.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On November 9, 1984, with the plant in cold shutdown, the Reactor Protection System [JC] low primary coolant flow trip was determined to be improperly set. Technical Specifications Table 2.3.1, Item 2, requires the low flow trip setpoint to be greater than or equal to 95% of the nominal four pump primary coolant flow. During the operating period from July 1984 to September 1984, the trip value did not meet this criteria.

The Reactor Protection System (RPS) low flow trip consists of four independent channels. Each channel utilizes a separate set of differential pressure (dp) detectors [PDT, JC] which measure steam generator dp.

Test Procedure T-69, "RPS Low Flow Trip Calibration", is used to determine the correlation between steam generator dp and total coolant flow. T-69 also determines the correct dp values for the low flow trip. These values are then used in Technical Specifications Surveillance Procedure MI-2, "Reactor Protection Trip Units", to verify the low flow trip occurs at the setpoint calculated in T-69.

Test Procedure T-69 was required to be performed in December 1981 during the plant startup following the 1981 refueling outage. A scheduling error by the plant support staff omitted this test. In addition, Surveillance Procedure MI-2 did not refer to T-69 to ensure the correct trip values were available.

An analysis was performed to determine the significance of the error in the low flow trip setpoint. The analysis has shown that the trip setpoint was within the Technical Specifications limit, except for the period of time from July 1984 to September 1984. During this time period, all low flow trip channels were found to be less than the required 95% setpoint. The greatest error was calculated to have been a setpoint of 94.37% using conservative calculations.

The basis provided for Technical Specifications Table 2.3.1 states that the low flow trip setpoint of 95% is required to ensure an RPS trip occurs at 93% after instrument error. The trip setpoint in all four channels remained well above the 93% value, and there were no challenges to the low flow trip. Consequently, no threat to public health or safety resulted.

Test Procedure T-69 was performed on November 15, 1984 during startup. The correct low flow trip values have been provided for Surveillance Procedure MI-2. To prevent recurrence of this error, requirements will be incorporated into Surveillance Procedure MI-2 to ensure T-69 is performed as required and that updated trip values are available and referenced.



**Consumers
Power
Company**

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US Nuclear Regulatory Commission
Document Control Desk
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DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 84-023
(REACTOR TRIP SET LESS THAN TECHNICAL SPECIFICATION LIMIT)

Attached please find Licensee Event Report 84-023 (Reactor Trip Set Less Than Technical Specification Limit) which is reportable to the NRC per 10 CFR 50.73(a)(2)(i), (a)(2)(ii), (a)(2)(v), and (a)(2)(vii).

Ralph R Frisch
Senior Licensing Analyst

CC Administrator, Region III, USNRC
Director, Office of Nuclear Reactor Regulation
NRC Resident Inspector - Palisades

Attachment