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Prairie Island Coalition
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Beth Wetzel
NRR/DRPW/PDIII-1
Mail Stop 13 D18
Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

Dear Ms. Wetzel:

2-19-96

Nuclear Information and Resource Service (NIRS) and Prairie Island Coalition (PIC), hereafter referred to as "Petitioners," hereby, file an addendum to the joint petition under 10 CFR 2.206 dated June 5, 1995 regarding intergranular stress corrosion cracking (IGSCC) of steam generator tubes at Northern State Power's Prairie Island nuclear power station Units 1 and 2.

In the June 5, 1995 petition to the Nuclear Regulatory Commission, the Petitioners specifically requested that "all steam generator tubes at Prairie Island Unit 1 and 2 be given a full length inspection" and "the Zetec Plus Point Probe and any state of the art, eddy current probe for corrosive cracking be employed." The Petitioners demanded that the operating license for Unit 1 and 2 be suspended until the requested action items be completed.

On January 31, 1996 in an open public session of the Periodic Briefing On Operating Reactors and Fuel Facilities, Mr. William Russell, Director of Nuclear Reactor Regulation, discussed steam generator performance issues focused on experience gained from the 1995 outages. Mr. Russell noted before the Commission that NRR had identified "cases of free span cracking" where cracking of steam generator tubes had been identified "not within the support plates or down at the tube sheet region."¹

It is the contention of the Petitioners that NSP never performed the full length inspections of all steam generator tubes with advanced eddy current testing equipment as requested by the emergency enforcement petition for neither the June 1995 outage at Prairie Island Unit 2 nor in the current outage for Unit 1.

The Petitioners further contend that as documented in the June 5, 1995 petition, Prairie Island Unit 1 has already experienced the rupture of a steam generator tube in 1979, releasing

¹ "Periodic Briefing on Operating Reactors and Fuel Facilities- Public Meeting," United States of America Nuclear Regulatory Commission, January 31, 1996, Transcript, p.65.

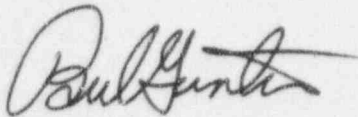
radiation into the atmosphere and that the risk of additional ruptures has only increased due to age related deterioration of the susceptible material that fabricates the unit's steam generators.

The Petitioners reassert their contention as presented in the June 5 petition that in terms of the probability of a steam generator tube accident, Thomas Murley, Director, Office of Nuclear Reactor Regulation, in a memorandum to James M. Taylor, NRC Executive Director of Operations, stated "that, currently, steam generator tube rupture events appear to be unavoidable."²

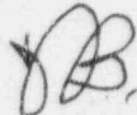
REQUESTED ACTION

Therefore, based on new information made available in the NRC Periodic Briefing dated January 31, 1996 and in an effort to protect the public health and safety and to preserve the quality of environment from radioactive contamination resulting from a steam generator tube rupture accident at the Prairie Island nuclear power station the Petitioners demand that the unit not be allowed to return to operation until the requested full length inspection of all steam generator tubes be conducted using the state of the art Zetec Point Plus Probe.

Sincerely,



Paul Gunter, Director
Reactor Watchdog Project
Nuclear Information and Resource Service
1424 16th Street NW Suite 404
Washington, DC 20036



Shean H. Bjoralt, Director
Communications and Logistics
Prairie Island Coalition
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Shirley Jackson, NRC Chair
Senator Paul Wellstone
NRC Office of the Inspector General
Joseph B. Campbell, Prairie Island Community
Office of Minnesota Attorney General

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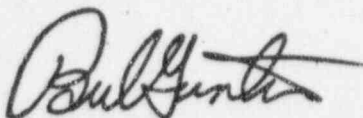
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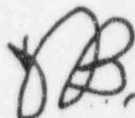
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Joseph B. Campbell, Prairie Island Community
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² Memorandum: Report on Trip To Sweden, France, NEA and the U.K.- June 18-30, 1992, Thomas E. Murley, US NRC, November 10, 1992.