

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 A L B R F 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

0	1
---	---

REPORT SOURCE 01 L 6 0 5 0 0 0 2 6 0 7 0 9 2 1 8 1 8 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During a refueling outage, the turbine to reactor building doors (No. 237 and
0 3 | 238) were opened simultaneously, breaching secondary containment. (T. S.
0 4 | 3.7.C). These doors connect the unit 2 reactor zone with the turbine bay.
0 5 | All required safety systems were available and operable. There was no effect
0 6 | on public health and safety.

0 7

[illegible]

Diagram illustrating the structure of the 20-bit data field:

SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE						COMP SUBCODE	VALUE SUBCODE							
0	9	S	A	(11)	E	(12)	X	(13)	X	X	X	X	X	(14)	X	(15)	Z	(16)
7	8	9	10	11	12	13	14	15	16	17	18	19	20					

(17) LER/RO REPORT NUMBER

EVENT YEAR

SEQUENTIAL REPORT NO.

OCCURRENCE CODE

REPORT TYPE

REVISION NO.

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS				ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER			
F	18	X	19	Z	20	Z	21	0	0	0	0	Y	23	N	24	L	25	X	9	9	9
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	The Folger Adams interlock system failed because of loose door latch screws
---	---	---

1 1 and electrical limit switches that were out of adjustment. The doors were

1	2	closed and repaired. Procedures were revised. Modified design was
---	---	---

1	3	successfully field tested. Implementation of the modified design is scheduled
---	---	---

for the unit 1 cycle 6 outage.

7	8	9	FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION		
1	5		H	28		0	0	0	29	NA		A	31	Operator Observation			
10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z 33 10 34 NA 45

PERSONNEL EXPOSURES

NUMBER		TYPE	DESCRIPTION
1	7	0 0 0 (37)	(38) NA

		PERSONNEL INJURIES					
NUMBER		DESCRIPTION					
1	8	0	0	0	(4)		NA 8412140460 841205

IE 22

7	8	9	10	11	12
LOSS OF OR DAMAGE TO FACILITY					
TYPE		DESCRIPTION			
L		L			

112178480 841205
 PDR ADOCK 05000260
 S PDR

1	9	Z	(42)						
7	8	9	10						

PUBLICITY

NRC USE ONLY

ISSUED		DESCRIPTION			
2	0	N	44	NA	

NAME OF PREPARER Stan D. Carter

PHONE: (205) 729-3891

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 82028R4 Technical Specification Involved 3.7.C

Reported Under Technical Specification 6.7.2.b.(2) * Date Due NRC

Event Narrative:

Unit 1 was operating normally at 98 percent; unit 3 was operating normally at 98 percent. These units were not affected by this event. Unit 2 was shut down for refueling outage and was affected by this event.

Equipment access doors (237 and 238) between the turbine building and unit 2 reactor building were observed open at the same time. An inspection revealed that the interlock system failed to operate properly and allowed both doors to be opened simultaneously.

The doors were padlocked and repairs were completed within four hours. The Folger Adams interlock system failed because of loose latch screws and electrical limit switches that were out of adjustment. All required safety systems were available and operable. There was no effect on public health and safety.

Procedures were revised to provide a closer tolerance for setting the door position limit switches.

A modified design which utilizes limit switches to detect the alignment of the latch and the electric strike was installed utilizing the plants special test procedure. Initial field evaluation of the results of this special test indicates that the modified design resulted in improved operation of the door interlock system. The modified design is scheduled to be installed on the personnel and equipment interlock doors for units 1, 2, and 3 during the unit 1 cycle 6 outage, which is expected to begin in June 1985.

This is also a followup report to LER BFRO-50-259/83008.

Previous Similar Events

BFRO-50-259/82063, 81008, 81050, 83008
260/80045, 81024, 81025, 81032, 81038, 77015

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

December 5, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

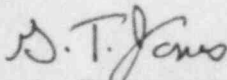
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-09 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/82028R4

The enclosed report provides additional details concerning breach of
secondary containment. This report is submitted in accordance with
Browns Ferry Unit 2 Technical Specification 6.7.2.b.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Resident Inspector, BFN

IE22
11