

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Prairie Island Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 8 2										PAGE (3) 1 OF 0 2																																																						
TITLE (4) Reactor Trip on Startup																																																																										
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																																																								
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)																																									
1			1			0			7			8			4			8			4			-			0			1			1			-			0			0			1			2			0			7			8			4			0 5 0 0 0						0 5 0 0 0					
OPERATING MODE (9) N						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																																																																				
POWER LEVEL (10) 0 2 4						20.402(b)						20.408(a)						X						80.73(a)(2)(iv)						73.71(b)																																												
						20.418(a)(1)(i)						80.38(a)(1)												80.73(a)(2)(v)						73.71(a)																																												
						20.408(a)(1)(ii)						80.38(a)(2)												80.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 365A)																																												
						20.408(a)(1)(iii)						80.73(a)(2)(i)												80.73(a)(2)(vii)(A)																																																		
						20.408(a)(1)(iv)						80.73(a)(2)(ii)												80.73(a)(2)(viii)(B)																																																		
						20.408(a)(1)(v)						80.73(a)(2)(iii)												80.73(a)(2)(ix)																																																		
LICENSEE CONTACT FOR THIS LER (12)																																																																										
NAME Arne A. Hunstad, Staff Engineer																								TELEPHONE NUMBER AREA CODE 6 1 2 3 8 8 1 1 1 2 1																																																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																																										
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC						CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC																																												
SUPPLEMENTAL REPORT EXPECTED (14)																								EXPECTED SUBMISSION DATE (15)						MONTH DAY YEAR																																												
YES (If yes, complete EXPECTED SUBMISSION DATE)																								X NO																																																		
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																																																										
During unit startup, a reactor trip occurred when the feedwater control system malfunctioned.																																																																										
Connector contacts were cleaned.																																																																										
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IE22 11																																																																										

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Prairie Island Unit 1	0 5 0 0 0 2 8 2 8 4	—	0 1 1	— 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On November 7, 1984 Unit 1 was returning to power following an outage. At 2026, with the unit at about 24% power, the operators were in the process of placing control of the main feedwater regulating valves (SJ) in automatic. When the "A" loop valve control was shifted to automatic, the controller (TC) output changed to minimum output, closing the valve. Before the control could be shifted back to manual and feedwater flow reestablished, automatic reactor trip occurred.

Although it was not apparent at the time of the event, further investigation revealed that the connector (CON) for either the "A" loop feedwater flow controller or steam generator level controller had a voltage drop across it. The cause of the voltage drop was dirty or oxidized contacts. The contacts were cleaned and the controller has been operating in automatic. Similar connectors will be cleaned at upcoming refueling outages of each unit.

Health and safety of the public were unaffected.



Northern States Power Company

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Minneapolis, Minnesota 55401
Telephone (612) 330-5500

December 7, 1984

U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket No. 50-282 License No. DPR-42

Reactor Trip On Startup

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72
on November 7, 1984.

for Eugene Eckholt
David Musolf
Manager - Nuclear Support Services

DMM/EFE/cab

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

IE22
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