LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

OCTOBER 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

OCTOBER 1-31

The Unit was shutdown for the entire month of October.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

   There were no facility license or Technical Specification Amendments during the reporting period.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER See Section II.C.	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L27305		RHR "C" Pump	High pump vibrations. Which could cause pump damage.	None: Pump still operational	Replaced original pump shaft & column with modified shaft.
L34618		VE Isolation damper to unit two	Damper continuously auto closes.	Inadequate ventilation in Aux. Elec. Equip. Room.	Removed damper actuator, sent out for repair and reinstalled.
L37048		VC Ventilation Damper	Damper drifts closed when"A" train is running	Redundant train still available	Damper actuator defective, sent out for repair and reinstalled
L40465		HPCS Suction Pressure gauge	Pressure gauge indicating zero pressure at pump suction.	Adequate suction pressure verified to be present via water leg pump discharge pressure gauge.	Recalibrated pressure gauge.
L40661		Drywell wiring	Provide support for drywell cable and equipment inspection.	Routine check for heat related degradation.	S & L inspection team assisted by CECO personnel as required.
L41921		Fuel Pool drain valve	Valve has packing leak which affects outcome of local leak rate tests.	Subject drain valve is not a containment boundary.	Replaced and tightened packing.
L42083		APRM B	Top row of APRM B down-scale lights not lit.	Redundant channels still available.	Changed out blown fuse.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L42159		Inboard Feed water check valve.	Subject check valve failed LLRT. Due to misalligned bushing wedging disc off seat.	Potential loss of cont-containment integrity.	Corrected bushing alignment and performed LLRT.
L42228		Diesel Fire pump batteries.	Specific gravities are low for pilot cells.	Low battery charge affecting start capabilities.	Charged batteries for 48 hours.
L42448		RHR SW PRM	"A" Loop PRM has continuous Hi-Rad signal above alarm setpoint.	Rad Chem samples show no contamination present in service water.	Disassembled PRM, cleaned and recalibrated.
L42930		Rx Water Level Instrument line.	Variable leg has leaking fitting.	Temporary loss of shutdown range.	Replaced segment of tubing and connectors.
L40210		I.S.H.I. Welds	Determine location of welds for hanger removal and inspection	Required for 1985 I.H.S.I. Inservice inspection.	Walked down RF and RH welds. Checklist generated.
L42048		Shutdown cooling Isolation Valve	Unable to open valve following unit shutdown.	Established alternate shutdown cooling mode.	Repaired valve operator.
L42176		Outboard feedwater check valve.	Valve Failed local Leak rate test.	Potential loss of primary containment integrity.	Repaired valve and retested.
L42999		RHR Heat exchanger outlet valve.	Valve needs limit switch adjustment.	Valve difficult to open.	Adjusted limit switch so that valve will not torque closed.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, October 1 through October 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
84-052-00	9/27/84	RWCU High Pump Room Differential Temperature Isolation.
84-053-00	9/16/84	RWCU High Differential Flow Isolation.
84-054-00	9/21/84	Reactor Core Isolation Cooling Isolation.
84-055-00	9/21/84	Reactor Water Cleanup Isolation.
84-056-00	9/21/84	Rx Scram on Group I Isolation While Performing LIS-MS-01.
84-057-00	10/1/84	Group I Isolation.
84-058-00	10/1/84	Start of VC/VE Emergency Makeup Train.
84-059-00	10/2/84	HPCS Discharge Relief Valve Bellows Seal Failure.
84-060-00	9/29/84	RCIC Steam Line High Differential Pressure Isolation.
84-061-00	10/1/84	LLRT of MSIV's Exceeding 100 SCFH.
84-062-00	9/30/84	Loss of Shutdown Cooling Due to 1E12-F009 Valve Failure.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE November 7, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: October 1984 GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	5377	5377
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	1165	1165
7.	HOURS GENERATOR ON LINE	0.0	5194	5194
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	1.0	1.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	0.0	14581709	14581709
10.	GROSS ELEC. ENERGY GENERATED (MWH)	0.0	4739789	4739789
11.	NET ELEC. ENERGY GENERATED (MWH)	0.0	4512808	4512808
12.	REACTOR SERVICE FACTOR	0.0	73.5%	73.5%
13.	REACTOR AVAILABILITY FACTOR	0.0	89.4%	89.4%
14.	UNIT SERVICE FACTOR	0.0	71.0%	71.0%
15.	UNIT AVAILABILITY FACTOR	0.0	71.0%	71.0%
16.	UNIT CAPACITY FACTOR (USING MDC)	0.0	59.5%	59.5%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	0.0	57.2%	57.2%
18.	UNIT FORCED OUTAGE RATE	0.0	17.1%	17.1%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:November 21, 1984
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		6/21/82
INITIAL ELECTRICITY		9/04/82
COMMERCIAL OPERATION		1/1/84

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: NOVEMBER 7, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761

MONTH: OCTOBER, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	0	17.	0
2.	0	18.	0
3	0	19.	0
4.	0	20.	0
5.	0	21.	0
6.	0	22.	0
7.	0	23.	0
8.	0	24.	0
9.	0	25.	0
10.		26.	0
11.		27.	0
12.		28.	0
13.		29.	0
14.		30.	0
15.	0	31.	0
16.	0		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle One
DATE November 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

REPORT MONTH OCTOBER 1984

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
20	84/09/29	S	744	H	4	Continuation of outage which started on September 29.

DOCUMENT 0044r/0005r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no relief valve actuations during the month of October.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-764-84	1B HPCS D/G	Recalibrate Instrumentation
1-775-84	1B HPCS D/G	Oil & Filter Change
1-789-84	HPCS Water Leg Pump	Lubrication
1-790-84	HPCS Pump	Discharge Relief Valve Replacement.
1-847-84	HPCS Pump & 1B D/G	Equipment Protection for LES-HP-02
1-873-84	"C" RHR Pump	Piping Struts Inoperable
1-887-84	1E12-F094 Valve	Cable Splice Repair
1-891-84	1B HPCS D/G	Circuit Breaker Inspection
1-898-84	HPCS Water Leg Pump	Lubrication
1-900-84	HPCS D/G Cooling Water Pump	Lubrication
1-924-84	LPCS Pump	Equipment Protection for LES-LP-01
1-952-84	1B D/G	Reenergize Bus 143
1-1029-84	1A D/G	Recalibrate Instrumentation
1-1036-84	RHR SW Pump	Lubrication
1-1069-84	LPCI Testable Check Valve	Disassemble and repair
1-1121-84	HPCS Pump	Discharge Relief Valve

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

Attached is a synopsis of the change that was made to the Radioactive waste treatment system during the month of October.

October 15, 1984

BACKGROUND

During the last few months, there have been many operational difficulties associated with the Stock Equipment System used to solidify Radwaste. Now that most of the difficulties have been resolved, radwaste solidification is proceeding in a timely manner. However, during the interim, a significant backlog of radwaste needing solidification has accumulated. This inventory hinders efficient operation of the radwaste system as a whole- i.e. limited available tank space, carryover in concentrators, and higher than desired tank dose rates.

In addition, new federal legislation on radwaste compacts goes into effect January 1, 1986. At that time, it may not be possible to send solidified waste to our present burial sites. Consequently, Commonwealth Edison has elected to be at zero inventory (empty tanks and no solidified barrels in storage on site) by January 1, 1986.

For the above reasons, it has been decided to employ NUS Process Services Corporation to solidify the contents of the 1A and 1B Phase Seperator Tanks, and the Waste Sludge Tank. The solidification and subsequent transportation of waste off-site is expected to begin October 22, 1984. Startup will require approximately two weeks, solidification/transportation will take six weeks, based on 3 weekly shipments and tear down will take one week.

PURPOSE

Our Stock System will have to be modified to allow interface with the Contractors equipment. These changes, although temporary, constitute major changes to the radwaste system, and need review per Technical Specification 6.9.1.

As the contractor will be using his own equipment, personnel, and procedures to perform the solidification, Technical Specification 4.11.3.1.b requires the solidification be performed in accordance with a Process Control Program to ensure the requirements of 10CFR61 are met. Technical Specification 6.7.1. requires approval by on-site review prior to the implementation at these procedures.

MODIFICATION DETAILS

All equipment is to be maintained within the Radwaste Truck Bay, with the exception of the cement truck, which will remain outside the Truck Bay. The initial solidification, mixing, curing, and inspection will take place in the Truck Bay, under Health Physics supervision. At that time, Health Physics personnel will determine, based upon area dose rates, and any airborne considerations, whether subsequent curing may proceed in a lined truck outside the Truck Bay. Entry level status of the Truck Bay will be upgraded at the discretion of the Health Physics personnel.

An interface to the existing drum processing unit will be needed to supply waste to the Contractor's equipment. Flanges on the existing transfer lines will be broken, and a manual valve with a Chicago fitting will be flanged in line. The vendor's supply hose will be attached to the Chicago fitting.

An interface to our ventilation system may be needed to handle equipment off-gas. A hose, with a HEPA filter attached, will be vented to the Truck Bay ventilation system.

An air supply, uncontaminated water supply, and electrical hookup are presently available in the Truck bay.

No liquid or gaseous radioactive material releases are expected. During the 1st solidification, samples will be drawn at the outlet of the liner vent HEPA filter to determine if any net release of radioactive material occurs, and if so, its extent. If there is any radioactive material measurable at the HEPA outlet, all subsequent solidifications will route the HEPA outlet directly to the Radwaste ventilation system - resulting in no release.

Increased exposures to individuals in the unrestricted area and to the general population are not expected. If curing does take place outdoors, dose rates at the boundary to the unrestricted area will be measured at this time to determine if any excess exposure to individuals at the boundary would occur None is expected.

An estimate of exposure to plant operating personnel as a result of the change will be performed by Rad/Chem personnel as standard procedure. Exposure will be estimated based on expected man-hours required for the flange modifications times measured dose rates observed prior to the modification.

Two days prior to the solidification, test solidifications will be performed by NUS. These tests will verify proper solidification of our waste at the chemical/cement concentrations prescribed in their Process Control Procedures. After a thorough recirculation of the tanks to mix the contents, one solidification of each of the phase seperators will be performed. Between three and five solidifications will be performed on the waste sludge tank, approximately one per week.

This document, the contractor procedures, and the 10CFR50.59 safety evaluation are being routed through onsite review prior to the commencement of the solidification, and will be included in the Semiannual Radioactive Effluent Release Report as detailed in Technical Specification 6.7.2, and in the NRC Monthly Operating Report as detailed in Technical Specifical 6.9.1.a. Both the reports require evidence that an on-site review of the changes was made.

Any comments or questions involving the above proposal should be addressed to JoAnn Shields, x330.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

OCTOBER 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

October 1-27

The Unit started the reporting period at 87% power. At 2038 hours on October 4, power was reduced to 60% due to heater drain flow oscillations. At 0700 hours on October 6, reactor power was raised to 86%. At 0700 hours on October 7, reactor power was raised to 98%. At 0450 hours on October 27, the reactor scrammed on an APRM Hi-Hi actuation caused by the reactor recirc flow control valves. The reactor was critical for 628 hours and 50 minutes.

October 28-31

The reactor went critical at 1947 hours on October 28. At 0103 hours on October 29, the main generator was synchronized to the grid. At 0700 hours on October 29, reactor power was raised to 48%. At 1433 hours on October 29, reactor power was raised to 72%. At 0000 hours on October 31, reactor power was raised to 85%. The reactor was critical for 76 hours and 13 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

   There were no amendments to the facility license or Technical Specification during the reporting period.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	LER See Section II.C.	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L38906		RCIC Drain stop valve	Light indication shows valve to be partially open with C/S in closed position.	Valve locally verified to cycle properly and fully closed with C/S in closed position.	Readjusted limit switch actuator arm.
L41991		CRD Low header pressure scram	Low charging header pressure setpoint not at required value.	Still provides trip but not at required setpoint.	Recalibrated trip set-point to Tech-Spec. value.
L42334		HPCS Water Leg pump pressure gage	Pump Discharge gauge indicating 100 psig.	HPCS piping locally verified to be filled and pressurized	Recalibrated pump discharge gauge.
L42489		ADS "S" accumulator pressure switch.	Low pressure alarm up for ADS "S" accumulator.	Locally verified N_2 pressure adequate and all other accumulator pressures adequate	Recalibrated pressure switch.
L42850		2A Diesel generator	Air start motors have 45 starts on them. Should be replaced.	Preventative maintenance measure to assure adequate start capabilities.	Air Start motors replaced.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, October 1 through October 31, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
84-063-00	9/1/84	RCIC Manually Isolated for Inop LD PCIS switch.
84-064-00	9/7/84	RWCU High Differential Flow Isolation.
84-065-00	9/24/84	RWCU High Ambient Temperature Isolations.
84-066-00	9/27/84	RWCU High Differential Flow Isolation.
84-067-00	9/27/84	RWCU Pump Room Differential Temperature Isolation.
84-068-00	9/6/84	HPCS System Inoperable.
84-069-00	9/12/84	Group I Isolation on Low Condenser Vacuum.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA RREPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE November 10, 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: October 1984 GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	705.1	273.0*	273.0*
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
7.	HOURS GENERATOR ON LINE	699.8	267.7	267.7
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2184610	817298	817298
10.	GROSS ELEC. ENERGY GENERATED (MWH)	723745	270373	270373
11.	NET ELEC. ENERGY GENERATED (MWH)	701050	222697	222697
12.	REACTOR SERVICE FACTOR	94.6%	87.5%	87.5%
13.	REACTOR AVAILABILITY FACTOR	94.6%	87.5%	87.5%
14.	UNIT SERVICE FACTOR	93.9%	85.8%	85.8%
15.	UNIT AVAILABILITY FACTOR	93.9%	85.8%	85.8%
16.	UNIT CAPACITY FACTOR (USING MDC)	90.8%	68.9%	68.9%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	87.3%	66.2%	66.2%
18.	UNIT FORCED OUTAGE RATE	6.1%	14.5%	14.5%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
There is an outage scheduled to begin on March 1, 1985 for maintenance and surveillances. This outage is expected to last approximately four weeks.
*Unit two was placed in commercial operation on October 19, 1984. Year-To-Date and cumulative totals are based on operating data starting October 19, 1984.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		3/10/84
INITIAL ELECTRICITY		4/20/84
COMMERCIAL OPERATION		10/19/84

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: November 10, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761
MONTH: October 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	1024	17.	1042
2.	1035	18.	1042
3	1041	19.	1065*
4.	1024	20.	1067
5.	749	21.	1067
6.	937	22.	1067
7.	1053	23.	1069
8.	1017	24.	1072
9.	1055	25.	1069
10.	1058	26.	1070
11.	1051	27.	202
12.	1049	28.	0
13.	1044	29.	548
14.	1046	30.	792
15.	1040	31.	832
16.	1000		

*October 19, 1984 marks the first date of commercial service.

ATTACHMENT R

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE November 1984
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

REPORT MONTH OCTOBER 1984

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
33	841004	F	0.0	A	5	Power reduction due to heater drain flow oscillations.
1	*841027	F	45.2	A	3	Reactor scram on APRM Hi-Hi actuation caused by the reactor recirc flow control.

* OCTOBER 19, 1984 MARKS THE FIRST DATE OF COMMERCIAL OPERATION.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

THERE WERE NO SAFETY RELIEF VALVE ACTIVATIONS DURING THE MONTH OF OCTOBER.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-1174-84	HPCS Pump	Discharge Relief Valve
2-1177-84	HPCS Pump	Discharge Relief Valve
2-1180-84	RHR Shutdown Cooling	Instrument surveillance
2-1191-84	RHR Heat Exchanger Outlet Valve	Set limit switches
2-1200-84	2A D/G	Replace air start motor
2-1204-84	2B D/G Air Compressor	Replace temperature switch

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

See report for Unit One for changes to radioactive waste treatment system.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

November 7, 1984

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering October 1 through October 31, 1984.

Very truly yours,

G. J. Diederich
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident
J. M. Nowicki, Asst. Comptroller

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