

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) BYRON, UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 4 5 4				PAGE (3) 1 OF 0 2		
TITLE (4) REACTOR TRIP																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES None				DOCKET NUMBER(S) 0 5 0 0 0 0			
0 3	0 8	8 5	8 5	0 3 1	0 0	0 4	0 1	8 5					0 5 0 0 0 0			
OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)														
POWER LEVEL (10) 0 1 1 8		20.402(b)				20.406(e)				<input checked="" type="checkbox"/> 90.73(a)(2)(iv)				73.71(b)		
		20.406(a)(1)(i)				90.38(a)(1)				<input type="checkbox"/> 90.73(a)(2)(v)				73.71(e)		
		20.406(a)(1)(ii)				90.38(a)(2)				<input type="checkbox"/> 90.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Test, NRC Form 306A)		
		20.406(a)(1)(iii)				90.73(a)(2)(i)				<input type="checkbox"/> 90.73(a)(2)(vii)(A)						
		20.406(a)(1)(iv)				90.73(a)(2)(ii)				<input type="checkbox"/> 90.73(a)(2)(vii)(B)						
		20.406(a)(1)(v)				90.73(a)(2)(iii)				<input type="checkbox"/> 90.73(a)(2)(viii)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Richard M. Williams, System Test Engineer, Ext. 385										TELEPHONE NUMBER AREA CODE 8 1 5 2 3 4 1 - 5 4 4 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS						
X				N												
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While operating in Mode 1 at a reactor power of 18%, a reactor trip occurred due to 1B Steam Generator LO-2 level. The low Steam Generator level was caused by the trip of the 1B Feedwater Pump, while personnel were investigating a lube oil problem. Following the Feedwater Pump trip, an unsuccessful attempt to start the Start-up Feedwater Pump was made. All instrumentation associated with the 1B Feedwater Pump and Start-up Feedwater Pump have been checked, and those instruments found out of tolerance have been recalibrated. These pumps will be continually monitored throughout start-up.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 5/31/86

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (3)

PAGE (3)

BYRON, UNIT 1

0 5 0 0 0 4 5 4

YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
85	03	1

0 2 OF 0 2

TEXT (if more space is required, use additional NRC Form 255A's) (17)

On March 8, 1985 at 1004 CST, Unit 1 was operating in Mode 1 at 18% power. The 1C Feedwater Pump and the 1A Motor Driven Feedwater Pump were out of service. The Start-up Feedwater Pump was shutdown and believed to be available.

Westinghouse representatives, noting two oil pumps were operating on the 1B Feedwater Pump Turbine, began troubleshooting the system (Only one oil pump is needed during normal operation). While performing a routine oil pump swapping maneuver, the Feedwater Pump tripped. An attempt was made to start the Start-up Feedwater Pump, but it failed to start. Auxiliary Feedwater was manually initiated to try and maintain Steam Generator level. Even though turbine and reactor power levels were reduced, a reactor trip occurred due to 1B Steam Generator LO-2 level. No Safety Injection occurred and the plant was stable in Hot Standby after 80 minutes into the event.

It was determined that the Start-up Feedwater Pump did not start due to an out of calibration oil pressure switch and a failed Snap-lock limit switch, both of which appear in the pump starting logic. The cause of the 1B Feedwater Pump trip has not been determined and has been declared indeterminate. All of its instrumentation has been recalibrated as a precaution.

This event has never happened before at Byron Station. The event posed no threat to public safety. The licensed personnel on shift and the Reactor Protection Systems performed appropriately to bring this event to a safe conclusion. Further action regarding the Feedwater Pump trip will be continued start-up monitoring and routine equipment checks.



**Commonwealth Edison**  
Byron Nuclear Station  
4450 North German Church Road  
Byron, Illinois 61010

April 1, 1985

LTR: BYRON 85-0471

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2)(iv) which requires a 30 day written report.

This report is number 85-031-00, docket No. 50-454.

Very truly yours,

R. E. Querio  
Station Superintendent  
Byron Nuclear Power Station

REQ/gt

Enclosure: Licensee Event Report No. 85-031-00

cc: J. G. Keppler, NRC Region III Administrator  
J. Hinds, NRC Resident Inspector  
INPO Record Center  
CECO Distribution List

#3/017

IE 22  
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