NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
ST. LUCIE UNIT 2	0 5 0 0 0 3 8 9	1 OF 0 1

TITLE (4)
INADVERTENT CONTAINMENT ISOLATION SIGNAL ACTUATION

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 1	0 4	8 4	8 4	0 0 9	0 0	1 2	0 4	8 4	St. Lucie 1	0 5 0 0 0 3 3 5
										0 5 0 0 0

OPERATING MODE (9)	6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
L. A. ROGERS, SHIFT TECHNICAL ADVISOR	AREA CODE 3 0 5 4 6 5 3 5 5 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
A	I K	M O N	G 0 6 3	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 4, 1984, while a scheduled refueling outage was in progress, a periodic calibration and testing of the containment radiation monitoring channels was being performed. Two-out-of-four radiation monitor channels in trip will cause a Containment Isolation Signal actuation of the Engineered Safety Features. Channel "C" was being tested and was in the tripped condition from a high radiation test signal. The control room operator was contacted by the I & C Test Engineer and was requested to reset the trip condition on channel "C". Immediately upon arrival at the Radiation Monitoring Control Board, the operator noticed a momentary "spike" on channel "A" (satisfying the two-out-of-four logic) and subsequent safeguards CIS actuation. All equipment actuated on a CIS functioned properly, (i.e., Unit 1 - Unit 2 control room recirculation, containment valve isolation, and diesel generator auto start.)

Investigation revealed no malfunctioned equipment in channel "A". No definite cause was identified as producing the "spike". However, welding was in progress in the area of the radiation monitoring equipment and could have caused the "spike".

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FLORIDA POWER & LIGHT COMPANY

December 4, 1984
L-84-364

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Gentlemen:

Re: Reportable Event 84-09
St. Lucie Unit 2
Date of Event: November 4, 1984
Inadvertent Containment Isolation
Signal Actuation

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,

J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/PLP/js

Attachment

cc: J. P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
PNS-LI-84-438-1

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