

TENNESSEE VALLEY AUTHORITY
DIVISION OF NUCLEAR POWER
SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT
TO THE
NUCLEAR REGULATORY COMMISSION
OCTOBER 1, 1984 - OCTOBER 31, 1984

UNIT 1

DOCKET NUMBER 50-327
LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328
LICENSE NUMBER DPR-79

Submitted by:

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for P. R. Wallace, Plant Manager

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Operations Summary

October 1984

The following summary describes the significant operational activities for the month of October. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 745.0 hours, produced 843,080 MWH (gross), resulting in an average hourly gross load of 1,131,651 kW during the month. There are 231.6 full power days estimated remaining until the end of cycle 3 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached July 31, 1985. The capacity factor for the month was 95.7 percent.

There were no reactor scrams or manual shutdowns, and one power reduction during the month.

Unit 2

The unit 2 cycle 2 refueling/modification outage continues. The estimated return-to-service date is November 30, 1984.

Significant Operational Events

Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
10/01/84	0001	The reactor was in mode 1 at 69% power and increasing. The unit load was 795 MWe.
10/17/84	0540	Condensate booster pump 1B tripped due to low oil level. The RO performed a manual runback to 60% reactor power.
	0545	Began power ascension.
10/18/84	0300	The reactor obtained 100% power.
10/31/84	2359	The reactor was in mode 1 at 100%, producing 1140 MWe.

Significant Operational Events

Unit 2

<u>Date</u>	<u>Time</u>	<u>Event</u>
10/01/84	0001	
10/31/84	2359	Cycle 2 refueling/modification outage continues.

PORVs and Safety Valves Summary

No PORVs or safety valves were challenged during the month.

Licensee Events and Special Reports

The following licensee event reports (LERs) were sent during October 1984 to the Nuclear Regulatory Commission.

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
1-84053	With unit 1 at 100 percent power and unit 2 in mode 3 (not standby), both trains of the auxiliary building gas treatment system were declared inoperable due to an uncontrolled breach of the ABSCE boundary. During a routine safety inspection of the auxiliary building and west valve room for unit 1, the inner door was noted to have been breached, using the proper administrative controls; however, the outer door was inadvertently left ajar allowing a breach of the ABSCE boundary. This configuration would have prevented the ABGTS from maintaining a minus 1/4-inch water gauge pressure with a vacuum relief flow of greater than 2000 CFM as required per technical specification 3.7.8, surveillance 4.7.8.d.3.
1-84057	<p>Inspections at Sequoyah Nuclear Plant have identified the following additional items of noncompliance with Appendix R of 10 CFR 50. These inspections are part of an ongoing project to ensure compliance with Appendix R.</p> <ol style="list-style-type: none">1. Power and local control cables for 1A, 1B, 2A, and 2B RHR pump room coolers interact such that all four (4) could be lost in a single fire. Cables involved are: 1PP3031A, 1PP3033A, 1PP3041B, 1PP3043B, 2PP3031A, 2PP3033A, 2PP3041B, and 2PP3043B.2. The A-A and B-B main control room air handling units and compressors have cables which interact such that a fire could take out both "A" and "B" trains. Cables involved are: 1PL4500A, 1PL4501A, 1PL4520A, 1PL4527A, 1PL4510B, 1PL4511B, 1PL4512B, 1PL4540B, 1PL4547B, 1PL4512B, 1PL4501A, 1PL4502A, 1PL4520A, and 1PL4527A.

Licensee Events and Special Reports (continued)

LER

DESCRIPTION OF EVENT

1-84057
(continued)

3. Cables which supply all four RCS loop temperature indications interact at one location on elevation 714 in the auxiliary building. Cables involved are:

1PV330K and 1PV320J.
4. Interactions exist between "B" train 6900 volt shutdown boards and cables for various "A" train valves. Valves and cables involved are:

2V1963A (2-FCV-3-136B), 2V1953A (2-FCV-3-136A),
2V2953A (2-FCV-3-116B), 2V2952A (2-FCV-3-116B),
2V2931A (2-FCV-116A), 2V2200A (2-FCV-67-125),
2V2201A (2-FCV-67-125), 2V2203A (2-FCV-3-125),
2V2211A (2-FCV-67-126), 2V2422A (2-FCV-67-146),
2V2423A (2-FCV-67-146), 2V2425A (2-FCV-67-146),
2V2968A (2-FCV-67-223), 2V2969A (2-FCV-67-223),
2V1983A (2-FCV-67-81), and 1V2425A (1-FCV-67-146).
5. Interactions exist between "A" train 480 volt shutdown boards and cables for various "B" train valves and "B" train ERCW pump cables. Cables involved are:

2V2941B (2-FCV-3-126A), 2V2961B (2-FCV-3-126B),
2V2083B (2-FCV-3-179A), 2V2131B (2-FCV-3-179B),
2V2807B (0-FCV-67-152), 2V2791B (0-FCV-67-152),
2V2794B (0-FCV-67-152), 2V2160B (2-FCV-67-123),
2V2792B (0-FCV-67-152), 2V2173B (2-FCV-67-124),
2V2160B (2-FCV-67-123), 2V2431B (2-FCV-67-147),
2V2170B (2-FCV-67-124), 2V2161B (2-FCV-67-123),
2V2429B (2-FCV-67-147), 2V2003B (2-FCV-67-82),
2PP704B, 2PP706B (ERCW Pump M-B), 2PP716B,
2PP718B (ERCW Pump P-B), 1PP704B, 1PP706B (ERCW
Pump L-B), 1PP716B, 1PP718B (ERCW Pump N-B).
6. Interactions exist between "B" train 480 shutdown boards and cables for various "A" train valves and "A" train components. The cables and components are listed below.

1V2951A (1-FCV-3-116B), 1V1953A (1-FCV-3-136A),
1V1960A (1-FCV-3-136B), 1V1983A (1-FCV-67-81),
1V2200A (1-FCV-67-125), 1V2211A (1-FCV-67-126),
1V2423A (1-FCV-67-146), 1V2431A (1-FCV-67-147),
1V2971A (1-FCV-67-223), 1V641A (1-FCV-67-424),
2V2971A (2-FCV-67-223), 2PLW227A (2-FCV-67-492),
1V2792A (0-FCV-67-151), 1PP679A (ERCW Pump J-A),
1V681A (ERCW Pump K-A), 1V691A (ERCW Pump R-A),
1V691A (ERCW Pump Q-A), 1PLW203A (A, ERCW Traveling
Water Screen), 2PLW211A (D, ERCW Traveling Water Screen).

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84057
(continued)

7. Interactions exist between train "A" 6900 volt shutdown boards and train "B" cables for valves and components. Involved cables are:

1V2941B (1-FCV-3-i26A), 1V2961B (1-FCV-3-126B), 1V2081B (1-FCV-3-179A), 1V2130B (1-FCV-3-179B), 1V2003B (1-FCV-67-82), 1V2163B (1-FCV-67-123), 1V2173B (1-FCV-67-124), 2V2428B (2-FCV-67-147), 2PLW233B (2-FCV-67-489), 1V3350B (1-FCV-67-478), 1PLW223B (1-FCV-67-489), 1PLW190B (ERCW Screen Wash Pump B), 2PLW180B (ERCW Screen Wash Pump C), 1PLW211B (ERCW Traveling Screen B), 2PLW203B (ERCW Traveling Screen C).
8. Unit 1 - In the auxiliary building on elevation 714 at A6Q and in the auxiliary control room on elevation 734 at RA6, the control cables for containment isolation valves, for normal letdown path, interact and fail to meet the 20-foot separation criteria required by Appendix R. Cables involved are:

1V4423A, 1V4435A, 1V4473A, 1V4488A, and 1V4503A.
9. Unit 1 - A fire at location A3 to A5/R to T on elevation 690 in auxiliary building could take out the following cables due to interactions with less than 20-foot separation.
 - (1) Train A: Handswitch for 1A-A AFW pump (cable 1PP652A) and 6900 volt to 1A-A AFW pump (cable 1PP650A).
 - (2) Train B: Handswitch for 1B-B AFW pump (cable 1PP664B) and 6900 volt to 1B-B AFW pump and 1B-B AFW pump and motor.
 - (3) Steam-driven AFW pump cables:

1SG461 and 1SG481 (LCV status light), 1PV105A (vital AC supply to 1-L-381), 1SG240A (vital AC to 669 transfer switch, normal), 1M811A (1-PS-3-138A), 1M1450A, 1M1452A (steam supply transfer), 1V2623A, 1V264A (1-FCV-1-17), 1PV185B (vital AC to 1-L-381), and 1SG221B, 1SG241B (alternate transfer switch).
10. Unit 1 and unit 2 - A fire at A12-A13/r line wall could involve the following "A" train components and "B" train reactor MOV board. Train "A" cables are:

1V2425A (1-FCV-67-146), 2V1955A (2-FCV-3-136A), 2V1965A (2-FCV-3-136B), 2V1985A (2-FCV-67-81), 2V2200A, 2V2201A, 2V2203A (2-FCV-67-125), 2V2211A (2-FCV-67-126), 2V2422A, 2V2423A, 2V2425A (2-FCV-67-146), 2V2931A (2-FCV-3-116A), 2V2951A, 2V2953A (2-FCV-3-116B), 2V2968A, 2V2969A (2-FCV-67-223).

Train "B" cables are reactor MOV board 2B2-B and associated cable trays.

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84057
(continued)

11. Unit 1 - A fire at location A3 to A10/Q to U on elevation 714 in the auxiliary building could involve cables for both motor-driven and turbine-driven auxiliary feedwater pumps along with their associated level control valves. (Listed below are the TDAFWP cables involved.)

1V3250A, 1V3251A, 1V3253A (1-FCV-1-15), 1M1452A, 1V3240A, 1V3243A, 1V3241A, 1M1450A, 1M1451A (1-FCV-1-16), 1V2624A, 1V2620A, 1V2621A, 1V2623A (1-FCV-1-17), 1V2633B, 1V2634B, 1V2635B (1-FCV-1-18), 1SG480, 1SG481, 1SG461, 1SG250S, 1SG251S, 1SG252S, 1SG229S (control cables, TDAFWP).

The action statement for technical specification 3.7.12 was satisfied by utilizing firewatches in the affected areas that were established by other Appendix R commitments. This action included the establishment of a roving fire watch in areas with fire detection and a dedicated fire watch in areas without fire detection.

Corrective Actions

An implementation schedule for corrective actions will be submitted to the NRC by January 1, 1985. The fire watches established will remain in effect until full compliance with the requirements of Appendix R can be achieved.

1-84058

A high radiation alarm was actuated at 0109 CST on September 17, 1984, while unit 1 was in mode 1, which caused a containment ventilation isolation (CVI) to occur. Investigation revealed that due to a leak in the manway cover on steam generator number one, steam entered the containment atmosphere and the resulting increase in moisture saturated the particulate filter and caused the iodine sample flow alarm to actuate. The contacts on the flow switch were very noisy and generated electromagnetic interference (EMI) which caused the high radiation alarm to actuate. There was no actual high radiation level in containment and no personnel were contaminated.

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84059

Inspections at Sequoyah Nuclear Plant have identified the following items of noncompliance with Appendix R of 10 CFR 50. These inspections are part of an ongoing project to ensure compliance with Appendix R.

1. Units 1 and 2 - Certain areas of the plant requiring operator action for the safe shutdown of the plant were surveyed using Abnormal Operating Instruction (AOI) 27, Control Room Inaccessibility. Areas requiring lights for operation of equipment specified by the shutdown logic are currently being evaluated. Also, the battery packs at Sequoyah are not rated for eight (8) hours.
2. The reactor coolant pump oil collection system does not fully comply with the requirements of 10 CFR 50, Appendix R, i.e., seismic design and capacity.

Each reactor coolant pump (RCP) holds 240 gallons of oil in the upper bearing system and 26 gallons in the lower bearing system. The RCP oil piping is seismically qualified (the piping installed on the pump), and the drain piping is Class 1 (L). Should one of the pump's oil systems fail, approximately 240 gallons of oil will be collected by the "pocket sump" and associated drain piping. The piping holds 140 gallons and the "pocket sump" holds 200 gallons. In the event that the "pocket sump" is full of water, the excess oil will overflow to the containment floor.

3. Inadequate separation (less than 20 feet) exists between the following valves:
 - a. Location - 480V transformer room 2B, valve 2-FCV-67-81 ("A" train) interacts with same system "B" train equipment.
 - b. Location - 480V transformer room 1A, valve 1-FCV-67-82 ("B" train) interacts with same system "A" train equipment.
 - c. Location - 480V transformer room 2A, valve 2-FCV-67-82 ("B" train) interacts with same system "A" train equipment.
 - d. Location - 480V shutdown board room, valve 2-FCV-67-489 ("B" train) interacts with same system "A" train equipment.
 - e. Location - Auxiliary control room, valve 1-FCV-67-147 ("A" train) interacts with same system "B" train equipment.
 - f. Essential raw cooling water (ERCW) pumps J-A, Q-A, K-A, R-A, and units 1 and 2 valve FCV-67-492 interact with 1, 2-FCV-67-489 ("B" train).

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84059
(continued)

- g. Location - Unit 1 penetration room, valve 1-FCV-67-81 ("A" train) interacts with 1-FCV-67-82 ("B" train).
 - h. Location - Unit 2 penetration room, valve 2-FCV-67-81 ("A" train) interacts with 2-FCV-67-82 ("B" train).
 - i. Valve 1-FCV-67-81 ("A" train) interacts with 1-FCV-67-82 ("B" train) down column line A1 through A6 and wall line Q through U, elevation 690.
 - j. Valve 2-FCV-67-81 ("A" train) interacts with 1, 2-FCV-67-82 ("B" train) down column line A11 through A15 and wall line Q through U, elevation 690.
 - k. Valves 1, 2-FCV-67-81 ("A" train) interact with 1, 2-FCV-67-82 ("B" train), elevation 669.
4. Less than 20-feet of separation exists, causing RHR pump 1A-A (cables 1PP575A, 1PP577A, 1PP578A, and 1PP580A) to interact with RHR pump 1B-B pump room cooler fan (cables 1PL4098B/1PL3043B, 1PL4097B/1PL3041B) on elevation 714, column A3/R.
 5. Less than 20-feet of separation exists between the RHR pump cooler 2A-A (cable 2PL3033A) and the 2B-B RHR pump (cables 2PP587B, 2PP589B, 2PP590B, and 2PP592B). Also, the 2A-A RHR pump cooler (cable 2PL3033A) interacts with 2B-B 6900V shutdown board on elevation 734, column A13/R.
 6. Less than 20-feet of separation exists, causing 1B-B RHR pump room cooler (cable 1PL3042B) to interact with the 1A-A RHR pump (cables 1PP575A, 1PP577A, 1PP578A, and 1PP580A) on elevation 734, column A3/Q.
 7. Less than 20-feet of separation exists, causing 2A-A RHR pump room cooler (cables 2PL3032A and 2PL3033A) to interact with 2B-B RHR pump (cables 2PP587B, 2PP589B, 2PP590B, and 2PP592B) on elevation 734, column A12/Q.
 8. Less than 20-feet of separation exists, causing 2A-A RHR pump room cooler (cables 2PL3031A and 2PL3033A) to interact with 2B-B RHR pump (cables 2PP587B, 2PP589B, 2PP590B, and 2PP592B) on elevation 714, column A13/R.
 9. Cables and conduit that provide level indication for the automatic level control valves (LCVs) on the motor-driven auxiliary feedwater pumps (MDAFWP) pass through a potential fire zone on elevation 714, column A6-A14/Q-U (conduit/cable: MC1099/2PM1223K; C226/2PM1232K, 2PM1241K, 2PM1251K). The level indication is required for manual level control.

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84059
(continued)

10. Less than 20-feet of separation exists, causing a common power cable for channel I temperature loops (2PV320J) to interact with a common power cable for channel II temperature loops (2PV330K) on elevation 714, column A4-A8/Q-R.
11. Less than 20-feet of separation exists between the two pressure loops P-68-69 and P-68-66. The cables are 2PV320J and 2PV135II, respectively, on elevation 714, column A4-A12/Q-R.
12. Less than 20-feet of separation exists between the two pressure loops P-68-69 and P-68-66. The cables are 2PM480I and 2PM481, respectively, on elevation 714, column A12-A14/Q-X and elevation 690, column A12-A13/X.
13. Cables for backup heater A-A pass on one side of a metal hatch and cables for backup heater B-B are on the other side of the metal hatch. The cables are 2PL4627A through 2PL4633A and 2PP820B, respectively, on elevations 749 and 734, column A11-A12/V. The hatch could conduct heat from a fire on elevation 734 to the control rod drive equipment room on elevation 749.
14. Unit 2 - In the auxiliary building on elevation 714 at A8/Q line, the 2A-A RHR pump cables interact with the 2B-B RHR pump cables and fail to meet the 20-foot separation criteria required by Appendix R. (Cables involved: 2PP578A, 2PP580A, 2PP590B, and 2PP592B.)
15. Unit 1 - In the auxiliary building on elevation 734 at A3/Q-R line; a fire in this area would require manual level control and the use of motor-driven AFW pump 1B-B. This condition also requires the use of the wide range level indication, whose cables pose an interaction of less than 20 feet. (Cables involved: 1PM1241K, 1PM1251K, and 1PM1223K for LI-3-56, -98, -111, and -43, respectively.)
16. Units 1 and 2 - In the auxiliary building on elevation 653 at A8/T line on either side of the elevator enclosure. The following RHR conduit and cables interact as they run from the floor to the ceiling on this elevation. (Cables involved: "A" train - 1PP575A (U1), 2PP575A (U2) interact with "B" train - 1PP587B (U1), 2PP587B (U2).

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84059
(continued)

17. Units 1 and 2 - In the auxiliary building on elevation 669 at A8/S to R line, the following cables for the "A" train RHR pump 1A-A interact with the "B" train RHR pump 1B-B. (Cables involved: "A" train - 1PP577A, 1PP575A and "B" train - 1PP589B, 1PP587B.) This same interaction applies to unit 2. (Cables involved: "A" train - 2PP575A, 2PP577A, and "B" train - 2PP587B, 2PP589B.)
18. Units 1 and 2 - On elevation 734 in the 6.9 kV shutdown board rooms, train "B" shutdown board interacts (less than 20-foot separation) with cables 1V2432A (1-FCV-67-147) and 1V2965A (1-FCV-67-223). For this interaction, it is assumed that the "B" train shutdown board is lost due to a fire. This fire would also cause 1-FCV-67-223 to fail, making the "A" component cooling heat exchanger inoperable. Also, 1-FCV-67-147 opening would crosstie the 2B and 1A essential raw cooling water headers.
19. Units 1 and 2 - On elevation 714 of the auxiliary building, please note the following table which indicates the "A" train - "B" train interaction (less than 20-foot separation) for the cables to the listed valves.

Unit 1

"A" Train

1-FCV-3-116A
1-FCV-3-116B
1-FCV-3-136A
1-FCV-3-136B
1-FCV-67-81
1-FCV-67-125
1-FCV-67-126
1-FCV-67-146
1-FCV-67-147
1-FCV-67-151
1-FCV-67-223
1-FCV-67-365
1-FCV-67-424
1-FCV-67-492
ERCW Pump Q-A
ERCW Pump J-A
Traveling Screen A

"B" Train

1-FCV-3-126A
1-FCV-3-126B
1-FCV-3-179A
1-FCV-3-179B
1-FCV-67-82
1-FCV-67-123
1-FCV-67-124
1-FCV-67-478
1-FCV-67-489
1-FCV-67-14
1-FCV-67-364
ERCW Pump L-B
ERCW Pump N-B
Traveling Screen B

Licenses Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84059
(continued)

19. (continued)

Unit 2

"A" Train

2-FCV-3-116A
2-FCV-3-116B
2-FCV-3-136A
2-FCV-3-136B
2-FCV-67-81
2-FCV-67-125
2-FCV-67-126
2-FCV-67-146
2-FCV-67-223
2-FCV-67-492
ERCW Pump R-A
ERCW Pump K-A
Traveling Screen D

"B" Train

2-FCV-3-126A
2-FCV-3-126B
2-FCV-3-179A
2-FCV-3-179B
2-FCV-67-82
2-FCV-67-123
2-FCV-67-124
2-FCV-67-147
0-FCV-67-152
2-FCV-67-489
ERCW Pump M-B
ERCW Pump P-B
Traveling Screen C

Corrective Action Taken or Planned

Item 1

Portable lighting has been supplied outside the main control room, by the shift engineer's office, to be used by the operators. Operators in the plant usually carry flashlights. Portable lighting is also supplied by the auxiliary control room in a cabinet by its entrance. TVA will reverify compliance with 10 CFR 50, Appendix R, Section J in accordance with the confirmation of action letter from O'Reilly to Parris dated August 10, 1984.

Item 2

System Operating Instruction (SOI) 55 was revised to include operator response to an RCP oil level alarm to pump down the sump if the level is greater than 45 percent. In addition, Surveillance Instruction (SI) 137.1 requires the pocket sump to be pumped down at least once per shift. TVA will reverify compliance with 10 CFR 50, Appendix R, Section III.0 in accordance with confirmation of action letter from O'Reilly to Parris dated August 10, 1984.

Items 3 through 19

The action statement for technical specification 3.7.12 was satisfied by utilizing fire watches in the affected areas that were established by other Appendix R commitments. This action included the establishment of a roving fire watch in areas with fire detection and a dedicated fire watch in areas without fire detection. An implementation schedule for corrective actions will be submitted to the NRC by January 1, 1985. The fire watches established will remain in effect until full compliance with the requirements of Appendix R can be achieved.

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84060

This report describes seven inadvertent auxiliary building isolations caused by spikes on the spent fuel pool radiation monitors. For all incidents, the spent fuel pool area was surveyed by Health Physics, and no indication of abnormal radiation levels were found. Operations personnel reset the signal, realigned all affected dampers to their normal position, and returned the ventilation systems to normal operation. The event dates included in this report are:

- (1) 1906 CST on September 21, 1984 from RM-90-102;
- (2) 1115 CST on September 23, 1984 from RM-90-102;
- (3) 1730 CST on September 24, 1984 from RM-90-102;
- (4) 0039 CST on September 28, 1984 from RM-90-102;
- (5) 0228 CST on September 28, 1984 from RM-90-102;
- (6) 2205 CST on September 29, 1984 from RM-90-103; and
- (7) 0114 CST on September 30, 1984 from RM-90-103.

The two spent fuel pool radiation monitors are Geiger-Mueller tubes manufactured by General Atomic (Models RD-1 and RP-1) mounted above the fuel pool and are designed to monitor the air spaces above the pool area. A high radiation signal initiates auxiliary building isolation and is designed to limit releases to the environment in the event of a fuel handling accident. For all events described, the auxiliary building isolations were caused from noise and/or background spikes reaching the monitor setpoint resulting in the automatic isolation. The range of these monitors is from 10^{-1} to 10^4 mr/hr, and the technical specification allowable setpoint is 15 mr/hr. The spikes causing the isolations peaked at approximately the setpoint value for the events described. The noise was from normal EMF on the monitor signal cable and the background radiation was from contaminated waste material passing near the monitors or fuel pool water radiation levels.

The following corrective actions have been taken to reduce the number of inadvertent auxiliary building isolations:

1. A one-second time delay was added to the high radiation isolation circuit to help discriminate between an inadvertent spike and an actual high radiation condition.
2. The spent fuel pool water was cleaned through its own filtering system to reduce the background levels to the monitors.
3. Contaminated waste material near the spent fuel pool monitors has been moved to reduce background levels and efforts are being made to maintain this area clear of contaminated waste.

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

1-84060
(continued)

4. A technical specification change is being pursued to increase the allowable isolation actuation setpoint from the present 15 mr/hr to a higher value.

It is expected that corrective actions one through three will limit the number of ABIs; however, until the technical specification setpoint change is obtained, inadvertent isolations from these monitors are expected to continue. For the events in this report, there was no effect upon public health and safety.

1-84061

On September 27, 1984, at 1219 CST, the reactor coolant system (RCS) pressurizer relief tank (PRT) pressure indicators located in the unit 1 and 2 auxiliary control rooms (ACRs) were discovered to have a range of 0-10 psig instead of 0-100 psig as required per technical specifications. The indicators were discovered to have the wrong pressure range while maintenance was being performed. The pressure indicators in the main control room read correctly. The unit one indicator was modified to read 0-100 psig and the unit two indicator will be modified during the present refueling outage.

1-84062

This LFR involves two separate incidents. The first control room isolation (CRI) occurred at 0930 CST on September 18, 1984 while unit 1 was in mode 1 (100 percent power, 2235 psig, 579 degrees F), and unit 2 was in mode 1 (100 percent power, 2235 psig, 578 degrees F). The CRI was reset and the chlorine detector was returned to service at 0935 CST on September 18, 1984. The second CRI occurred at 2310 CST on September 29, 1984 while unit 1 was in mode 1 (30 percent power, 2235 psig, 557 degrees F), and unit 2 was in mode 4 (0 percent power, 375 psig, 332 degrees F). The CRI was reset at 2320 CST on September 29, 1984. The radiation monitor (RM) chart drive motor was repaired at 2320 CST on October 10, 1984. The RM was operable during this time.

In the first incident, an assistant unit operator (AUO) pushed the test button on the chlorine detector without first placing the chlorine analyzer in the test position. The AUO was attempting to correct the drip rate on the detector. The AUO immediately realized what had happened and notified the unit operator to reset the CRI and realign the ventilation system. Instrument Maintenance was notified to correct the drip rate on the detector and verify the detector's operability. No problem was found with the chlorine detector. Instructions are written and a warning signal is posted to warn personnel about this type of error. Personnel failed to heed the warning in this instance. A yellow caution sign is being added to the detector that is more visible and gives a more explicit warning to personnel.

Licensee Events and Special Reports (Continued)

LER

DESCRIPTION OF EVENT

- 1-84062
(continued) In the second incident, the high radiation alarm on the radiation monitor (RM) was actuated by a spike. The spike was caused by the generation of electromagnetic interference (EMI) when the recorder chart drive motor was moved. The recorder is periodically pulled out from the RM to change the chart paper. The EMI was generated due to a lead on a capacitor being loose. The loose capacitor lead caused an intermittent connection when the recorder was moved. The operation of the RM was not affected as long as the recorder was not moved. The capacitor lead was resoldered to correct the EMI problem. No other problem was found, and the RM was returned to service.
- 1-84064 While in mode 5 (cold shutdown) at 2148 CST on September 25, 1984, and with reactor trip breakers closed, the unit experienced a trip on low-low steam generator level on loop 1. The main steam isolation valve was cycled for testing, releasing pressure on the steam generator, resulting in a swell, then a shrinkage, of level on the secondary side. This effect had not been anticipated before the test. Steam generator level was recovered after the trip, due to automatic start of the auxiliary feedwater.
- 2-84015 Unit 2 was in mode 1 (2235 psig, 579 degrees F) at 100 percent reactor power on September 5, 1984. At 0523C, the operator received a turbine generator electrohydraulic control system (EHC) high-low level alarm. An assistant unit operator was immediately dispatched to the EHC tank to investigate the problem. At 0529C, the operator received an EHC low oil pressure alarm. At 0530C, the reserve EHC pump automatically started. At 0532C, the generator turbine tripped on ECH low-low pressure and level resulting in an automatic reactor trip.
- 2-84016 On September 9, 1984 at 0803C, a reactor trip occurred due to a turbine trip which was caused by the generator neutral over-voltage relay actuating. The relay operation was verified as valid, and a megger test of the generator, isolated phase bus, and main and unit station service transformers indicated a ground on the system. The ground was found to be caused by a neoprene gasket/isolating strip that had fallen down onto the isolated phase (IPB). All the strips in the IPB were inspected, removed, and reinstalled with RTV as an adhesive. A preventive maintenance program is being implemented to inspect the isolated phase bus at each refueling outage. The unit stabilized at 547 degrees F following the reactor trip.

Licensee Events and Special Reports (Continued)

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
2-84017	During startup following recovery from a reactor trip (see SQRO-50-328/84015), unit 2 experienced another reactor trip. Just prior to the event which occurred at 1000C on September 6, 1984, unit 2 was in mode 1 (2235 psig, 551 degrees F) at 22 percent reactor power. The balance of plant operator was in the process of switching control of steam generator levels from manual to automatic. During the switchover, the number three (3) steam generator level began increasing and at 60 percent level feedwater was isolated to loop 3. The reactor operator reduced reactor power in an attempt to slow the swell of the number 3 steam generator. With reactor power at 18 percent, the steam generator level shrink due to the feedwater isolation resulted in a low-low level in the number four (4) steam generator, which tripped the reactor. The unit stabilized at 547 degrees F following the reactor trip.
2-84018	On September 10, 1984, three reactor trips occurred due to steam generator low-low level. These occurred at 0412 CST, 0901 CST, and 1253 CST, and all involved operators attempting to manually control steam generator levels on the feedwater bypass valves while trying to transfer to automatic controls.

Diesel Generator Failure Reports

There were no diesel generator failure reports transmitted during the month.

Special Reports

The following special reports were transmitted during the month.

<u>SPECIAL REPORT NUMBER</u>	<u>DESCRIPTION</u>
84-04	A penetration fire barrier breaching permit was issued on August 16, 1984 for fire door A-33 due to the door panel splitting at the welded seam. This door is located on elevation 669 between the refueling water purification pump and filter area (safety-related area) and the cask decon collector tank, pump, and filter area (nonsafety-related). The cause of the split panel has been attributed to hard closing forces due to the high differential pressure across the door.

Licensee Events and Special Reports (Continued)

SPECIAL
REPORT
NUMBER

DESCRIPTION

84-05 The biological shield blocks have been removed from unit 2 reactor building equipment hatch for the unit 2 refueling outage. This condition has been identified as a breach of a fire barrier penetration and will remain nonfunctional for the duration of the refueling outage.

84-06 Event Description Door A-92

Door A-92 was found to be inoperable by a fire watch on October 7, 1984. Penetration fire barrier breach permit number 1830 was written and the controls of technical specification 3.7.12 were placed in effect. Maintenance request (MR) number A121506 was written and sent to be loaded into the MR tracking system. The MR was not loaded into the system, and no work was performed on the door during the seven days that breach permit number 1830 was in effect. On October 14, 1984, permit number 1830 expired, and a second permit, number 1907, was written along with a new MR, number A121508. The craftsmen received the MR on October 15, 1984, and the door was repaired the same day. A postmaintenance surveillance was performed to verify operability of the door and breaching permit number 1907 was closed out.

Event Description Door DE-3

On September 24, 1984 at 2140 C.T, the penetration fire barrier breach permit (PFBBP) for door DE-3 expired. The door had been breached to allow ventilation to the CDWE building since the air conditioning unit was inoperable. A fire watch had been established and remained in effect throughout the door breach. On September 26, 1984 at 1030 CST, an audit of the breach permit book by an ASE revealed the permit had expired. The door was immediately closed.

Offsite Dose Calculation Manual Changes

There were no changes to the Sequoyah Nuclear Plant ODCM this month.

OPERATING DATA REPORT

DOCKET NO. 50-327
 DATE NOVEMBER 8 1984
 COMPLETED BY M. G. EDDINGS
 TELEPHONE (615) 870-6248

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1
 2. REPORT PERIOD: OCTOBER 1984
 3. LICENSED THERMAL POWER(MWT): 3411.0
 4. NAMEPLATE RATING (GROSS MWE): 1220.6
 5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: _____

 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): _____

 10. REASONS FOR RESTRICTIONS, IF ANY: _____

NOTES:

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745.00	7320.00	29257.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	745.00	4742.10	19183.66
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	745.00	4531.80	18644.95
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2509115.23	14213187.07	59705037.37
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	843080.00	4662940.00	20042076.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	814661.00	4472070.00	19248998.00
19. UNIT SERVICE FACTOR	100.00	61.91	63.73
20. UNIT AVAILABILITY FACTOR	100.00	61.91	63.73
21. UNIT CAPACITY FACTOR (USING MDC NET)	95.25	53.22	57.31
22. UNIT CAPACITY FACTOR (USING DER NET)	95.25	53.22	57.31
23. UNIT FORCED OUTAGE RATE	0.00	23.94	19.50
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	_____		
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	_____		

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327
 UNIT Sequoyah One
 DATE November 1, 1984
 COMPLETED BY M. G. Eddings
 TELEPHONE (615) 870-6248

MONTH OCTOBER 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1017</u>	17	<u>931</u>
2	<u>1104</u>	18	<u>1089</u>
3	<u>1084</u>	19	<u>1101</u>
4	<u>1105</u>	20	<u>1104</u>
5	<u>1107</u>	21	<u>1103</u>
6	<u>1105</u>	22	<u>1101</u>
7	<u>1106</u>	23	<u>1103</u>
8	<u>1104</u>	24	<u>1105</u>
9	<u>1105</u>	25	<u>1093</u>
10	<u>1106</u>	26	<u>1102</u>
11	<u>1104</u>	27	<u>1097</u>
12	<u>1102</u>	28	<u>1100</u>
13	<u>1103</u>	29	<u>1098</u>
14	<u>1105</u>	30	<u>1099</u>
15	<u>1103</u>	31	<u>1100</u>
16	<u>1102</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
 UNIT NAME Sequoyah One
 DATE November 6, 1984
 COMPLETED BY M. G. Eddings
 TELEPHONE (615) 870-6248

REPORT MONTH OCTOBER 1984

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
15	841017	F	N/A	A	5	.			Manual runback to 60%. Condensate booster pump tripped on low oil level.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Cont. of Existing
 Outage
 5-Reduction
 9-Other

⁴
 Exhibit G-Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I-Same Source

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE NOVEMBER 7, 1984
COMPLETED BY D.C. DUPREE
TELEPHONE (615) 870-6248

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: OCTOBER 1-31, 1984
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: _____
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): _____
10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745.00	7320.00	21217.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	6124.75	16485.82
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	0.00	5987.99	16142.31
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	19449577.30	51867645.11
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	6620740.00	17652680.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	6373689.00	16991426.60
19. UNIT SERVICE FACTOR	0.00	81.80	76.08
20. UNIT AVAILABILITY FACTOR	0.00	81.80	76.08
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.00	75.85	69.76
22. UNIT CAPACITY FACTOR (USING DER NET)	0.00	75.85	69.76
23. UNIT FORCED OUTAGE RATE	0.00	7.42	8.59
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			
November 30, 1984			

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-328
UNIT	Sequoyah Two
DATE	November 6, 1984
COMPLETED BY	D. C. Dupree
TELEPHONE	(615) 870-6248

MONTH OCTOBER 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328

UNIT NAME Sequoyah Two

DATE November 6, 1984

COMPLETED BY D. C. Dupree

TELEPHONE (615) 870-6248

REPORT MONTH OCTOBER 1984

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
13	840928	S	745	C	4				Refueling outage.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing
Outage
5-Reduction
9-Other

4
Exhibit G-Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

5
Exhibit I-Same Source

(9/77)

Plant Maintenance Summary

The plant maintenance summary for significant maintenance items completed during the month of October 1984 are listed in the following order:

Construction Activities
Electrical Maintenance Section
Electrical Modification Section
Instrument Maintenance Section
Mechanical Maintenance Section
Mechanical Modification Section

Construction Activities

ECN L5599 - Fifth Vital Battery

The project is approximately 55 percent complete. The fifth vital battery board has been placed in the battery room through a hole cut in the concrete roof. Work continued to set the batteries and the board, installation of fire protection piping and HVAC fans. Electrical tie-ins to battery boards I and III have been completed, and preparations are being made to make tie-ins to the other two boards during the unit 2 cycle 2 outage.

ECN L5841 - Hot Machine Shop

The project is 99 percent complete. Health Physics laboratory, deconning machines, electric shop, snubber shop, hot machine shop are in use during unit 2 cycle 2 outage. Work continued on communication and fire detection cable pulling and equipment installation. Monorail and hoist over electro-polishing equipment in the decon room will be done later when material and design drawings are available.

ECN L6182 - Cooling Tower Repair

Custodis Ecodyne - The contractor finished ice damage related work on both cooling towers and left the site. They will come back the latter part of November to complete the remaining two punchlist items (concrete louvers and nozzles).

ECN L5503, 5111 - Office and Power Stores Facility

The project is approximately 90 percent completed. During this month, work continued on pulling cable, insulating ductwork and piping, architectural painting, and drywall installation. Work was completed on the east entrance-way, all fans were installed, hydrostatic testing of fire protection piping was completed, and permanent power cable was pulled to the building. The installation of toilet fixtures, toilet tile work, and electrical connection of heat pumps and air handling units has begun.

ECN L5609, 5610 - Makeup Water Treatment Building

The project is approximately 68 percent completed. During this month, work continued on installing TVA-supplied piping, cable trays, protective coating application, and yard piping. Work was completed on the replacement of damaged instrumentation tubing on vendor-supplied equipment. Heat-traced process piping was started in order to expedite as-built drawing preparation which is necessary prior to ordering heat trace cables.

ELECTRICAL MAINTENANCE SECTION

DATE....	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
10-16-84	0-FAN-030	FUEL HANDLING EXHAUST FANS WILL NOT TRIP UNTIL RELAY ON HI RAD MONITOR RESETS	HI-RAD MONITOR (0-RM-90-103) OPERATES RELAY WHICH AUTOMATICALLY RESETS WHEN HI-RAD IS REMOVED THUS THE FUEL HANDLING EXHAUST FANS WILL NOT TRIP UNTIL RELAYS RESET	ADJUSTED TRIP UNTIL SET POINT WAS ACHIEVED. RESET HI-RAD AND LEFT OPERATING NORMALLY	A294545
10-16-84	1-CHR-313-0433 -B	1B-B 480VOLT BOARD ROOM CHILLER IS NOT COOLING LIKE IT SHOULD	LOOSE ADJUSTMENT BOLTS	TIGHTENED ADJUSTMENT BOLTS AND ADJUSTED HEAD PRESSURE	A234140
10-25-84	2-ZS-063-0064	LIMIT SWITCH OUT OF ADJUSTMENT CAUSING THE HAND SWITCH IN CONTROL ROOM TO SHOW BOTH RED AND GREEN LIGHTS	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	A294343
10-25-84	0-LOCL-013-063 0	ALARM KEEPS COMING IN ON PYTRONICS PANEL (PRIMARY POWER FAILURE PANEL RELAYS DISABLED)	RESET ON ALARMS OUT OF ADJUSTMENT	CHECKED DURING PERFORMANCE OF SI 234.7 SYSTEM IN NORMAL CONDITION RESET ALL ALARMS	A294461
10-25-84	1-RE-090-0120	MOTOR ON STEAM GENERATOR BUILDING LTQ SAMPLE MONITOR IS ROTATING BACKWARD	MONITOR HAD NO FLOW	CHANGED MOTOR ROTATION USING M&AI 12 PERFORMED SI 206 CLEANED ROTOR METER AND SAMPLE CHAMBER LEFT IN SERVICE	A121955
10-25-84	1-RE-090-0121	MOTOR ON STEAM GENERATOR BUILDING LTQ SAMPLE MONITOR IS ROTATING BACKWARD	MONITOR HAD NO FLOW	CHANGED MOTOR ROTATION USING M&AI 12 PERFORMED SI 206 CLEANED ROTOMETER AND SAMPLE CHAMBER LEFT IN SERVICE	A121956
10-25-84	0-RM-090-0212- A	ON STATION SUMP MONITOR HI RAD ALARM AND INSTRUMENT MALE ALARM WILL COME IN AND CLEAR. THEN FEW MINUTES LATER	BAD MOTOR	REPLACED MOTOR CLEANED FLOW SWITCH AND LEFT IN SERVICE	A286870

ELECTRICAL MAINTENANCE SECTION

DATE....	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
		IT WILL COME IN AGAIN 212 SUCTION VALVE WILL NOT OPEN PROPERLY RESTRICTING FLOW			
10-29-84	2-BCTA-072-001 O-B	TIMER ON CONTAINMENT SPRAY PUMP 2B-B CAN NOT BE ADJUSTED	AGASTAT TIMER BAD	REPLACED AGASTAT TIMER AS REQUESTED AND SET TIMER PER SURVEILLANCE INSTRUCTIONS 220.2	A232741
10-29-84	1-FCV-072-0013 -A	LOOSE CONNECTION ON BUCHANAN PLUG ON CONTROL ROOM HAND SWITCH	LOOSE CONNECTION	TIGHTENED AND FUNCTIONALLY TESTED CONTROL ROOM HAND SWITCH	A293544
10-29-84	1-HST-079	UNIT 2 MANIPULATOR CRANE AUXILIARY HOIST WILL NOT WORK	NOT KNOWN AT THIS TIME	BORROWED UNIT 1 MANIPULATOR CRANE AUXILIARY HOIST FOR USE ON UNIT 2 MANIPULATOR CRANE DURING U2C2 OUTAGE DOCUMENT ON TACF 1-84-105-79 HOLD ORDER 1411	A246787
10-29-84	O-RLY-026-0FPS	WHILE TESTING FIR DETECTORS IN THE LAUNDRY AREA 690 FLOW SWITCH RELAY FAILED TO RESET	RELAY FAILED TO RESET	CHECKED FPS RELAY AND FPS TIMER RELAY AND PS 235 RELAY ALL CHECKED OK WE FUNCTIONALLY TESTED ZONE 349 & 350 ALL RELAYS RESET PROPERLY	A284821
10-29-84	2-0XF-068-0341 -F	PRESSURIZER HEATER DOES NOT LOAD PROPERLY TO MAINTAIN PRESSURE	SCREEN DIRTY AND FUSE BLOWN	REPLACED SCREEN AND FUSE CHECKED LOAD ON 6.9 K.V. BREAKER AND IS BALANCED AT 26 AMPS PER PHASE (374 AMPS PER PHASE AT 480 VOLTS)	A293662
10-29-84	O-XS-013-0144- D	SMOKE DETECTOR NEAR PIPE GALLERY BEHIND ELEVATOR WILL NOT WORK	BAD DETECTOR	REPLACED SMOKE DETECTOR	A288596
10-29-84	2-HST-079	MANIPULATOR CRANE	UNKNOWN AT THIS TIME	INSTALLED AUXILIARY HOIST	A246788

DATE....	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
		AUXILIARY HOIST HAS BEEN REMOVED FROM UNIT 2		ON UNIT 2 THAT WAS BORROWED FROM UNIT 1 ON MRA246787	
10-29-84	2-BCTB-030-003 8-A	AMPECTOR WAS BAD IN CIRCUIT BREAKER (SB. 2A1-A COUP LOC) OF SHUT DOWN BOARD. THE CURVE WAS FIGURED WRONG ON SI258.2 THE IMPUT CURRENT WAS TOO LOW	AMPECTOR BAD	REPLACED AMPECTOR IN WESTINGHOUSE BREAKER. WHEN THE CURVE WAS REFIGURED BREAKER 30-38A WAS APPLIED THE PROPER CURRENT AND PASSED THE REQUIREMENTS OF SI 258.2	A234147
10-29-84	2-LOCL-013-063 0	UNABLE TO RESET FCV-26-223 DUE TO ALARM IN FROM ZONE 374B	FCV-26-223 WILL NOT RESET	REPLACED DETECTOR ON ZONE 374B RAN SI 234.7 ON ZONE 374 CHECKED OPERATION AND RETURNED TO SERVICE	A295005
10-31-84	2-CRN-079	LIMIT SWITCH ON MANIPULATOR CRANE NOT MAKING CONTACT	LIMIT SWITCH ARM WAS ACCIDENTLY TRIPPED	RESET LIMIT SWITCH ARM	A285730

17 records listed.

Electrical Modification Section

October 1984

DCR 1739 - VAACS

Panel wiring has continued this period at a slow pace. Material shortages still impede completion of this work. The Public Safety Service will meet November 1 with field engineers to establish the availability of remaining material needs.

DCR 2072 - Local DCR to Improve Reliability of Stud Tensioner Hoists

The original limit switches were replaced with geared limit switches. The 480-volt connectors were not available to be installed in this time period. The hoists were hard-wired in place. If the connectors arrive before the end of the outage, they will be installed. In any case, the stud tensioner hoists will be removed before the end of the outage and stored outside of containment.

ECN 2768 - RVLIS

All conduit inside and outside containment was completed during this period. All rework associated with this ECN was completed. All cable inside and outside containment was pulled. Terminations are underway inside and outside containment. All main control room work is complete with the exception of installing four level indicators, which are expected to be onsite in early November.

ECN 2780 - Postaccident Sampling Facility

All electrical modification work associated with the ventilation system has been completed. Communication equipment in the facility is nearing completion. At the present time, there are four valves that are not wired pending resolution of leak-rate testing deficiencies. Installation of heat trace has begun and is expected to be completed in early November. All lighting is complete.

ECN 5194 - Iodine Monitors

All electrical work associated with the monitor was completed early this period. The monitor has been analyzed and turned over to Instrumentation for initial checkout. Work remaining consists of keycard readers in both PING rooms. The workplan is still in the approval cycle.

ECN 5198 - Technical Support Center

The Ayden display generator was received onsite and installed early this period. All conduit work was completed this period. Approximately 95 percent of the cable has been pulled, and terminations are underway. The old SMS cabinet was removed from the main control room and is being reworked to accommodate the SPDS monitor. Work is on schedule, and there are no anticipated delays.

Electrical Modification Section (Continued)

ECN 5640 - Deletion of PDIS 1-17 and 1-18

The instruments were removed and associated pipes capped. All electrical work is complete.

ECN 5645 - Steam Generator Blowdown

All conduit and cables have been completed on this ECN. Terminations are in progress in the main control room and in local panels. The system is expected to be ready for postmodification tests early in November.

ECN 5712 - Evacuation Alarms

Conduit and cable have been installed in the unit 2 letdown heat exchanger room. Conduit has been installed in the east valve room, and 90 percent of the conduit has been installed in the west valve room.

ECN 5765 - Deletion of Seal-in Limit Switches

All work was completed in this period for this ECN.

ECN 5770 - High-Range Radiation Monitor

Work completed this period consisted of remaining panel wiring in 2-M-30. All work is complete.

ECN 5823 - Pressure Switch Replacement

Four pressure switches were replaced this period, for a total of ten associated with unit 2. There are four more pressure switches on order, and they are expected to arrive onsite October 31. They will be installed in early November.

ECN 5824 - Replacement of Valve Operators

Thirty-four valve operators were replaced this period. Four more operators remain on this ECN.

ECN 5842 - Removal of PCV-3-122 and PCV-3-132

Electrical work this period consisted of lifting leads on a maintenance request to facilitate removal of the valves. Workplan 11239 is awaiting information from the Instrument Maintenance Section. This information is expected to be finalized shortly, releasing the workplan for work.

ECN 5865 - Relocation of LA-77-129

All conduit and cable associated with this ECN are complete. We are presently waiting until the coolant system is brought back down below elevation 693 to remove the annunciator from service for relocation.

Electrical Modification Section (Continued)

ECN 5867 - Fuel Transfer System

All work was completed on this ECN at the beginning of this period. Postmodification testing was performed and found to be satisfactory. However, during initial fuel movement, the Red Lion counter continually reset during the transfer of fuel. Westinghouse was notified of this problem and recommended two field changes be made. The FCNs were implemented, and the transfer system has since performed satisfactorily.

ECN 5881 - Limit Switch Replacement

Twenty-eight of fifty-two limit switches have been replaced this period. There are no anticipated problems at this time.

ECN 5882 - Temperature Switch Replacement

Twenty-two System 30 temperature switches have been replaced and are functional at this time. Eighteen System 12 temperature switches have been installed in the field. Field wiring is complete. We anticipate final connection to the control circuit in early November.

ECN 5883 - Replacement of Various Instruments

Conduit and cable associated with the relocation of four flow elements and flow switches have been completed. Terminations are underway. Conduit and cable work associated with the relocation of six pressure switches has begun. Work involving the direct replacement of nine pressure switches will commence in early November.

ECN 5884 - Replacement of Flow Transmitters

Work has commenced to replace five System 3 flow transmitters. This is expected to be completed this period. Two System 72 flow transmitters required redesign of mounting brackets. Approval of this redesign is in hand, and the replacement of these flow transmitters is expected to start this week.

ECN 5898 - Deletion of Seal-in Limit Switches

Cable pulling for this ECN is complete. Workplan 11227 was PORC'd on October 30. Installation of relays and wiring changes will commence this week. This work involves five valves.

ECN 5970 - Replacement of Valve Operators

Two System 72 valves are waiting for final pipe relocation to facilitate their replacement. The four feedwater isolation valves have arrived onsite. Planning is underway to determine the most efficient way of delivering the valves to the worksites. There are no anticipated problems at this time.

Electrical Modification Section (Continued)

ECN 5971 - Replacement of Valve Operators

Replacement of valve 63-172 started late in the period. It is expected to be back in service by October 31. The two PORV block valves had their operators changed out early in the period.

ECN 5995 - Replacement of Various Instruments

The four PDTs have been replaced, and limit switches have been replaced on three valves. All field work on this ECN is complete.

ECN 6018 - Installation of Space Heaters, Auxiliary Feedwater Pump Motors

At the present time, we are waiting for a redesign of the installation of the heaters. The original design called for boring two 3-inch holes into the motor casing. We are requesting a design that will incorporate the use of vent openings to provide access for the heaters.

ECN 6032 - Relocation of Hydrogen Analyzers

All conduit and cable work is complete on this ECN. Trains of analyzers have been turned over to Instrumentation for functional testing.

ECN 6053 - Replacement of Acoustic Monitor Charge Converters

The new converters have been mounted, and conduit work is in progress. There are no anticipated delays.

ECN 6055 - Fourth Wide-Range Pressure Transmitter

Conduit and cable work inside containment is nearing completion.

ECN 6108 - Replacement of Valve Operators

This ECN has been superseded by ECN 6245. We have received instructions on modifying a unit 1 valve operator to replace the unit 2 valve operator. The writing of the workplan is in progress.

ECN 6200 - Relocation of Pressure Transmitters

Conduit and cable are approximately 50 percent complete on this ECN. The six pressure transmitters and their associated panels have been relocated.

ECN 6204 - Installation of Fuses

The majority of this work is on hold pending technical specification revision. The work that is available will commence in early November.

Electrical Modification Section (Continued)

ECN 6207 - Conax Connectors

Workplans involving other ECNs that require Conax connectors have been revised to incorporate the use of the connectors. The remaining devices that require the connectors will be addressed on a separate workplan, which is now in the approval cycle. There are no delays expected.

Appendix R

Work continues on the first five interactions.

Instrument Maintenance Section

October 1984

Unit 1

1. The unit experienced a turbine runback to 60 percent when 1B condensate booster pump tripped on October 17, 1984 during a transient with the level in the hotwells. It was found that the NPSH pressure switch had drifted high causing an inadvertent actuation during the transient. The switch was replaced, and the unit returned to full power.
2. During the monthly calibration of the UHI level switches, one switch was found outside of technical specification tolerance.
3. The TACF on the pressurizer relief tank remote monitoring pressure channel, 1-PT-68-311C, was cleared. In September, the channel was changed from 0-10 PSI to 0-100 PSI to agree with technical specifications. The ECN and drawing were received this month to clear the TACF. The modification is being implemented on unit 2 during the current refueling outage.

Unit 2

1. The unit has been in cycle 2 refueling outage this month. Response time testing and refueling calibration continue. Modifications for installation of TSC, RVLIS, new post accident radiation monitors, and harsh environment instruments to meet 10 CFR 50.49 requirements have been started.

Other work is shown on the attached list.

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A040293	1	RM	090	121		10/05/84	1-RM-090-121, FLUSH MONITOR TO REDUCE BACKGROUND.	CONTAMINATED SAMPLE CHAMBER. FLUSHED WITH RAW SERVICE WATER & RAN SI-206 FOR CAL
A096162	2	LT	077	411		10/15/84	2-LT-077-411, XMTR RESPONSE IS IMPAIRED DUE TO SUSPECTED LOSE OF FILL FLUID IN SENSING ELEMENT.	TRANSMITTER LOW ON FILL FLUID. REFILLED TRANSMITTER & VERIFIED CALIBRATION.
A233390	2	FCV	062	93		10/09/84	2-FCV-062-93, CHECK STROKE OF VALVE	CHARGING HEADER FLOW CONTROL VLV STROKE WAS ALTERED BY MAINTENANCE PERFORMED ON A PREVIOUS MR. VLV WAS RESTROKED.
A233392	1	TI	083	5001		10/29/84	1-TI-083-5001, NO INDICATION-CHECK OUT AND REPAIR	BAD CABLES. REPAIRED BY ELECTR PER MR #234150.
A234140	1	CHR	313	433		10/11/84	1-CHR-313-433, *I* INVESTIGATE AND RESOLVE PROBLEMS WITH 4800 BD ROOM CHILLER 18-B	LOOSE ADJUSTMENT BOLTS. ADJUSTED HEAD PRESSURE(TIGHTENED ADJUSTMENT BOLTS)
A234146	0	BATD	250			10/16/84	0-BATD-250-, *I* PREPARE THE NEW 5TH VITAL BATTERY FOR INITIAL EQUALIZE CHARGE	NO FAILURE. PREPARE THE NEW 5TH VITAL BATTERY FOR INITIAL EQUALIZER CHARGE. CHECKED BATTERY CONNECTIONS, VERIFIED WATER LEVEL TO BE 1" ABOVE CELL PLATES AND LABEL CELLS 1 THROUGH 60.
A237705	2	MTRA	003	0128		10/11/84	2-MTRA-003-0128, REPLACE LUBRICATING OIL ON AUXILLARY FEEDWATER PUMP MOTOR.	NO FAILURE. REPLACE LUBRICATING OIL ON AUXILARY FEEDWATER PMP MOTOR B. DRAINED AND REFILLED WITH 4 QUARTS OF STO-2 OIL
A237708	2	MTRA	003	0118		10/11/84	2-MTRA-003-0118, REDPLACE LUBRICATING OIL ON AUXILLIARY FEEDWATER PUMP MOTOR.	NO FAILURE. REPLACE LUBRICATING OIL ON AUXILARY FEEDWATER PMP MOTOR A. DRAINED AND REFILLED WITH 4 QUARTS OF STO-2 OIL
A239443	0	RM	090	102		10/01/84	0-RM-090-102, *I* CHECK CALIBRATION OF MONITOR.	THE FUEL POOL RADIATION MONITOR WAS SHOWING HIGHER ACTIVITY THAN HP SURVEY. INVESTIGATED AND FOUND NO APPARENT PROBLEM.
A282234	2	FCV	062	93		10/28/84	2-FCV-062-93, VERIFY 2-FCV-62-93 IS STROKING PROPERLY FROM CONTROLLER	POSITIONER OUT OF CAL., VLV OUT OF STROKE. CALIBRATED POSITIONER, STROKED VLV.
A285082	2	HGR	074	483		10/12/84	2-HGR-074-483, HANGER 2-RHRH-483 - PAINT WELDS AND HANGER AS REQUIRED	NO FAILURE. HANGER WELDS CORRODED AND NEEDS PAINTING. CLEARED CORROSION FROM HANGER WELDS ETC AND PAINTED BLACK
A285084	2	HGR	074	492		10/12/84	2-HGR-074-492, HANGER 2-RHRH-492 - PAINT WELDS AND HANGER AS REQUIRED	NO FAILURE. HANGER WELDS CORRODED AND NEEDS PAINTING. CLEARED CORROSION FROM HANGER WELDS ETC AND PAINTED BLACK
A285086	2	HGR	012	8-14		10/12/84	2-HGR-012-8-14, HANGER 47A431-8-14 - PAINT WELDS AND HANGER AS REQUIRED	NO FAILURE. HANGER WELDS CORRODED AND NEEDS PAINTING. CLEARED CORROSION FROM HANGER WELDS ETC AND PAINTED BLACK
A285097	2	HGR	062	416		10/12/84	2-HGR-062-416, HANGER 2-CVCMH-416 - WELDS NEEDS PAINTING	NO FAILURE. HANGER WELDS CORRODED & NEEDS PAINTING. CLEARED CORROSION FROM

COMP

MR.COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A285899	1	LIC	003	174	10/15/84	1-LIC-003-174, *I* PLEASE CALIBRATE ABOVE MENTIONED LIC	HANGER, WELDS ETC AND PAINTED BLACK PROBLEM WITH SERVO MOTOR & VARIABLE RESISTOR. REPLACED CONTROLLER WITH SPARE.
A288557	2	MTRA	062	108A	10/15/84	2-MTRA-062-108A, 2A CENTRIFUGAL CHARGING PUMP MOTOR - ADD OIL TO OUTBOARD MOTOR BEARINGS.	NO FAILURE. OIL ON OUTBOARD MOTOR BEARINGS ON 2A CENTRIFUGAL CHARGING PMP MOTOR. ADDED 1/4 PINT OF STO-5 OIL TO OUTBOARD MOTOR BEARINGS
A288684	0	CHR	313	725-B	10/17/84	0-CHR-313-725-B, B TRAIN 69 SHUTDOWN BD. ROOM CHILLER-HIGH MOISTURE CONTENT IN FREON	NO FAILURE-INDICATOR LIGHT SHOWING CAUTION ON B TRAIN 6.9 KV SHUTDOWN BOARD RM CHILLER(HIGH MOISTURE CONTENT). CHECKED AS REQUESTED, FOUND TO BE IN GOOD OPERATION, SIGHT GLASS CLEAR AND ALL PRESS. FOUND TO BE FINE. NO WORK PERFORMED
A291755	0	RM	090	125	10/11/84	0-RM-090-125, INVESTIGATE SPIKING PROBLEM ON WHICH LEADS TO CONTROL ROOM BLDG VENT ISOLATION	LEAD ON CAPICATOR BROKEN. RESOLDERED LEAD
A292811	2	HGR	001	310	10/15/84	2-HGR-001-310, RUSTY SNUBBER	NO FAILURE. RUSTY SNUBBER FOUND IN WEST VLV RM. CLEANED AND PAINTED SNUBBER PER MI 10.14
A292831	2	HGR	001	10	10/15/84	2-HGR-001-10, HANGER NEEDS PAINTING	NO FAILURE. HANGER NEEDS PAINTING IN WEST VALVE RM. CLEANED & PAINTED HANGER AS REQUESTED.
A293505	1	RM	090	120	10/05/84	1-RM-090-120, FLUSH OUT MONITORS TO REDUCE BACKGROUND AND CLEAR ALARMS.	CONTAMINATED SAMPLE CHAMBER. FLUSHED WITH RAW SERVICE WATER & RAN SI-206 FOR CAL
A293519	0	RM	090	122	10/07/84	0-RM-090-122, CHEMICALLY CLEAN RAD MONITOR TO REDUCE BACKGROUND TO <10 TO 5TH POWER CPM	SAMPLE CHAMBER CONTAMINATED. CLEAN SAMPLE CHAMBER & VERIFY CAL
A293680	2	FCV	074	24A	10/15/84	2-FCV-074-24A, THIS VLV CYCLES OPEN(PARTIALLY) WHILE ON RHR B TRAIN; FLOW CONDITION IS >2500GPM UNTIL VLV OPENS UP. THEN DROPS BELOW 2500GPM(TECH SPEC)	SWITCH OUT OF CAL. RECALIBRATED SWITCH.
A293715	0	RM	090	212	10/06/84	0-RM-090-212, "INSTRUMENT MALFUNCTION ALARM" WILL NOT CLEAR. FLOW DRUM STAYING IN WHEN SUMP PUMP IN SERVICE.	FLOW SWITCH STUCK. CLEANED SWITCH & FLUSHED MONITOR, PERFORMED SI-206
A293716	2	FM	030	273A	10/12/84	2-FM-030-273A, *I* THE FM IS NOT FOLLOWING A.B. PRESS. WILL NOT MAINTAIN A.B. PRESS. IN NORMAL RANGE	FAULTY DELTA P TRANSMITTER. REPLACE TRANSMITTER.
A294256	2	MTRA	074	0010-A	10/04/84	2-MTRA-074-0010-A, THE UPPER SITE GLASS ON 2AA RHR PUMP NEEDS OIL ADDED	NO FAILURE. OIL LEVEL LOW. ADDED OIL TO UPPER SIGHT GLASS ON 2A-A RHR PMP 1 PINT OF STO-2

COMP

MR. COMP U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

A294520	2 PT	070 17A	10/01/84	2-PT-370-17A. PRESS SWITCH STARTS CCW PUMPS THAT AP_ IN AUTO BEFORE HEADER PRESS APPROACHES 40 PSIG	COMPONENT COOLING SYSTEM PUMPS 2A-A AND 2B-B DISCHARGE LOW PRESS. SWITCH WAS OUT OF CALIB. COMPONENT COOLING SYSTEM PUMPS 2A-A AND 2B-B DISCHARGE LOW PRESS. SWITCH WAS RECALIB.
A294542	0 RM	090 132	10/09/84	0-RM-090-132, #1# FLOW LOW WON'T CLEAR, HAD ALL FILTERS CHANGED-STILL WOULD NOT CLEAR	FILTER PAPER NOT INSTALLED PROPERLY & LID ON FILTER BOX NOT TIGHTENED DOWN. INSTALLED PAPER CORRECTLY & TIGHTENED FILTER BOX LID.
A295006	0 BATB	250	10/15/84	0-BATB-250-, #1# SPECIFIC GRAVITY READING 1.195 THIS IS T.S. LIMIT	NO FAILRE. CHECK SP. GRAVITY OF CELL, IF BELOW 1200 PLACE ON EQUALIZER CHARGE. IF SP. GRAVITY IS BELOW 1195, REPORT IMMEDIATELY TO SE D/G 2B-B BATTERY. PLACED BATTERY 2B-B OF DIESEL GENERATOR ON 7 DAY EQUALIZER CHARGE.

29 records listed.

Mechanical Maintenance Section

October 1984

1. ERCW scab plate work on the third and fourth locations was completed, thereby completing this project.
2. The boric acid transfer pump seal was replaced.
3. The 2B1 MSR tube bundle was pulled and replaced with a stainless steel bundle.
4. Pinhole leaks on the ERCW piping at 1-FCV-67-146 above the CS heat exchanger resulted in the installation of a scabplate.
5. Monthly inspections of 1A-A and 2A-A diesel generators were completed per SI-102.
6. FCV-1-18 was threaded and capped due to steam blowing from the gland leak-off line.
7. The floor drain collector tank line was drilled and drained due to blockage in the line.

Mechanical Modification Section

October 1984

ECN 6047

A temporary reactor vessel level indicator for unit 2 was installed and is operable.

DCR 2032

The reactor coolant pump seal leak-off lines were capped.

ECN 5773, Pressurizer Work, ECN 6196

Final designs were received for all but three hangers. All three pressurizer safeties were replaced with three modified valves. The Nu-con insulation for the loop seals is on site.

NJREG 0588

ECN 6231

Several piping reroutes on the fire protection, CVCS and waste disposal systems were completed allowing the new motor operators to be installed.

ECN 6032 - Relocate H₂ Analyzer

This work has been completed except for postmodification testing.

ECN 5895 - Solenoid Replacement

All but five solenoids have been replaced.

ECN 5457 - Solenoid Replacement

Only one solenoid remains to be replaced.

ECN 5883 - Auxiliary Feedwater Pressure Switches

Work continues on the relocation of these switches.

ECN 6200 - Auxiliary Feedwater Transmitter Relocation

The instruments have been relocated. The repiping work has not started.

Other Work

Mechanical support was provided to the Electrical Section in the replacement of the 0588 instruments.

Mechanical Modification Section (Continued)

ECN 5644 - Auto Dump Back to the Condenser

Some painting and insulation activities remain.

ECN 5645 - SGBD

Painting, insulation, and final testing will be performed during startup.

ECN 5770 - Turbine Building Radiation Monitor

Final tie-ins were made and postmodification testing has begun.

ECN 6002 - CST Check Valves

The diafloat check valves have been installed.

ECN 5024 - Steam Generator Wet Layup

SGBD tie-ins were completed. The feedwater bypass check valves and the discharge tie-in were completed. Work remaining includes tie-ins to the chemical feed lines, hanger work, painting, and insulation.

ECN 5842 - Cavitating Venturi

All tie-ins have been completed. Hydro testing is in progress and hanger work continues.

ECN 6060

The upper internal lift rig platform was installed prior to floodup.

ECN 5534

The under vessel access cage was installed and painted.

ECN 5751

Penetration X117 was installed.

ECN 5713 - Paul Monroe Snubber Monorail

Work continues. Two steam generator enclosures are complete except for painting.

DCR 1935 - Turbine Generator Platform

The turbine generator platform has been installed on unit 2.

ECN 5743 - Steam Generator Platforms and Ladders

This work is 90 percent complete.

Mechanical Modification Section (Continued)

DCR 1729 - MSR Doghouse HVAC

Work continues in this area. Work is being coordinated with other MSR work.

ECN 6243 - Penetration X87

The workplan was written and approved.

ECN 6206 - CST B

The tank has been sandblasted and the first coat of paint applied. Application of the final coat remains. The Williams Company HVAC equipment is being used to maintain the temperature and humidity within the allowable limits during work.

ECN 6256, 6257, 6262 - Turbine Building Side Performance Improvement Modifications

Workplans have been written and are in the approval cycle. Material is being located.

ECN 2768 - RVLIS

All containment unistrut and sensor installation was completed. Some paint work remains. All capillary tubing was installed. Tie-ins to the hot legs and the reactor head remain prior to pressure testing and postmodification testing. Westinghouse personnel are on site to assist in these activities.

ECN 2780 - Post Accident Sampling

All penetration isolation valves were installed and tested. All valves are being procured from Watts Bar Nuclear Plant. Tubing work inside containment and in the PASF room continues. Postmodification testing was started on the HVAC and parts of the tubing.

ECN 5939 - MFPT Condenser Retubing

The condensers were opened, old tubes removed, and new tubes installed. Harris Tube Pulling and Foster Wheeler have completed their work. The first tube is presently being reinstalled. Both expansion joints have been replaced.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

NOV 14 1984

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

SEQUOYAH NUCLEAR PLANT - MONTHLY OPERATING REPORT - OCTOBER 1984

Enclosed is the October 1984 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P. R. Wallace
P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

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