

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Nine Mile Point Unit #1DOCKET NUMBER (2)  
0 5 0 0 0 0 2 2 0PAGE (3)  
1 OF 0 2

TITLE (4)

Reactor Scram While Performing Surveillance Test N1-ISP-ATWS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)											
0	3	0	4	8	5	8	5	0	0	3	0	0	0	4	8	5	0	5	0	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)													
POWER LEVEL (10)	1	0	0	20.402(b)			20.406(c)			<input checked="" type="checkbox"/> 80.73(a)(2)(iv)			73.71(b)		
				20.406(a)(1)(i)			80.36(a)(1)			<input type="checkbox"/> 80.73(a)(2)(v)			73.71(c)		
				20.406(a)(1)(ii)			80.36(a)(2)			<input type="checkbox"/> 80.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)		
				20.406(a)(1)(iii)			80.73(a)(2)(i)			<input type="checkbox"/> 80.73(a)(2)(vii)(A)					
				20.406(a)(1)(iv)			80.73(a)(2)(ii)			<input type="checkbox"/> 80.73(a)(2)(vii)(B)					
20.406(a)(1)(v)			80.73(a)(2)(iii)			<input type="checkbox"/> 80.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)

NAME  
Robert Randall, Supervisor, Technical SupportTELEPHONE NUMBER  
AREA CODE  
3 1 5 3 4 9 - 2 4 4 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs
A									

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NOEXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

## ABSTRACT

While at full power, monthly surveillance test N1-ISP-ATWS was being performed. Technicians performing this test tried to rectify an error condition in a reactor level transmitter. While manipulating valves a reactor scram was initiated.

Procedure N1-ISP-ATWS has been revised to caution technicians to the affect of valving errors on other transmitters on the sensing line.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Nine Mile Point Unit #1	0 5 0 0 0 2 2 0	8 5	- 0 0 3	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

TEXT

On March 4, 1985, while at 100% power, Instrument and Control technicians were performing surveillance procedure N1-ISP-ATWS. Level transmitter LT-36-21A was valved out of service and connections were made per procedural requirements. Initial readings indicated a gross voltage error. At about 14:34, to eliminate this error, technicians involved began purging air from the transmitter, manifold assembly, and associated tubing. While manipulating valves to purge air from the system an inadvertent, momentary, water hammer was introduced into the transmitter's low-side. The water hammer also appeared on the reference header, and was transmitted to all devices connected to this header. Two extremely sensitive pieces of instrumentation which monitor reactor level and provide RPS trips for reactor low-level scram and turbine trip are connected to this header. These are transmitters 36-03A (RPS Channel A) and 36-03B (RPS Channel B). This water-hammer amounting to a one pound surge was sufficient to initiate the low level channel 11 and channel 12 safety systems which resulted in a full scram.

ASSESSMENT OF POTENTIAL SAFETY CONSEQUENCES

There are no potential safety consequences arising out of this event because all of the engineered safety features involved in this event operated as designed; therefore, there was no damage to the plant or danger to the plant personnel arising out of this event.

CORRECTIVE ACTION

Procedure N1-ISP-ATWS has been revised to caution technicians to the affect of valving errors on other transmitters on the sensing lines.

## NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST  
SYRACUSE, N. Y. 13202

April 3, 1985

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555RE: Docket No. 50-220  
LER 85-03

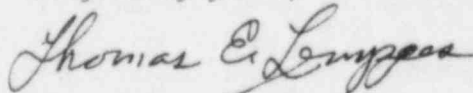
Gentlemen:

In accordance with 10 CFR 50.73, we hereby submit the following  
Licensee Event Report:

LER 85-03      Which is being submitted in accordance with  
10 CFR 50.73 (a) (2) (iv), "Any event or  
condition that resulted in manual or automatic  
actuation of any Engineered Safety Feature (ESF),  
including the Reactor Protection System (RPS).  
However, actuation of an ESF, including the RPS,  
that resulted from and was part of the preplanned  
sequence during testing or reactor operation need  
not be reported."

This report was completed in the format designated in NUREG-1022,  
dated September 1983.

Very truly yours,

Thomas E. Lempges  
Vice President  
Nuclear GenerationTEL/lo  
attachmentscc: Dr. Thomas E. Murley  
Regional AdministratorLE22  
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