

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Washington Nuclear Plant - Unit 2

DOCKET NUMBER (2)

05000397

PAGE (3)

1 OF 012

TITLE (4)
Containment Temperature MonitoringEVENT DATE (5)
MONTH DAY YEAR
* 8 4 - 0 3 4 - 0 4 1 1 8 4

LER NUMBER (6)

REPORT DATE (7)

OTHER FACILITIES INVOLVED (8)

FACILITY NAMES
DOCKET NUMBER(S)
050000OPERATING MODE (9)
POWER LEVEL (10)
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)
20.402(b) 20.405(c) 50.73(a)(2)(iv) 73.71(b)
20.405(a)(1)(i) 50.38(a)(1) 50.73(a)(2)(v) 73.71(c)
20.405(a)(1)(ii) 50.38(a)(2) 50.73(a)(2)(vii) X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii) 50.73(a)(2)(i) 50.73(a)(2)(viii)(A)
20.405(a)(1)(iv) 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B)
20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(ix)
Special Report
Tech. Spec. 3.7.8.aLICENSEE CONTACT FOR THIS LER (12)
NAME
R. L. Koenigs, Compliance Engineer

TELEPHONE NUMBER

AREA CODE

509377-125011

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)
Ext. 2279
CAUSE SYSTEM COMPONENT MANUFAC. REPORTABLE TO NRC
B M B - - - - - NSUPPLEMENTAL REPORT EXPECTED (14)
X YES (If yes, complete EXPECTED SUBMISSION DATE) NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR
06 30 85

ABSTRACT (Limit to 400 spaces - add only free single-space typewritten lines) (16)

During Plant heatups Primary Containment Monitoring System (CMS) temperature indicators exceeded 150°F for more than 8 hours on 4/14/84 through 4/15/84, 4/28/84 through 4/29/84, 7/3/84 through 7/11/84, 8/4/84 through 8/6/84, 9/18/84 through 9/19/84 and 11/3/84 through 11/8/84. During each event the average drywell temperature did not exceed 135°F (as per Technical Specification 3.7.8a). The 11/3-8/84 condition is a continuation of the original problems addressed in Revisions 0, 1, 2 and 3 of this LER.

*	Date	Operating Mode	Power Level	LER No.
Event 1	4/14-15/84	2	001	84-034 Rev. 0
Event 2	4/28-29/84	2	001	84-034 Rev. 0
Event 3	7/3-11/84	1	035	84-034 Rev. 1
Event 4	8/4-6/84	1	050	84-034 Rev. 2
Event 5	9/18-19/84	1	045	84-034 Rev. 3
Event 6	11/3-8/84	1	090	84-034 Rev. 4

8412110094 841129
PDR ADOCK 05000397
S PDR122
111

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7 8 4	—	0 3 4	—	0 4 0 2	OF 0 2	

TEXT (If more space is required, use additional NRC Form 385A's) (17)

Plant Condition

Event	Plant Mode	Power Level	Affected Thermocouple
1	2	1%	CMS-TE-51, 54, 55 & 18
2	2	1%	CMS-TE-51, 54, 55 & 20
3	1	35%	CMS-TE-55
4	1	50%	CMS-TE-55
5	1	45%	CMS-TE-26, 32
6	1	90%	CMS-TE-19, 51

Events

All events were reportable per Technical Specification 3.7.8.a. Event 1 initiated a redesign of the drywell ducting. Event 2 and all subsequent events were resultant from incomplete ducting modifications to the drywell cooling units CRA-FN-1A, 1B, 1C, 2A & 2B. 150°F was exceeded intermittently over the 11/3 - 11/8/84 period. Further modifications to ducting and drywell insulation repair and installation are still forthcoming. Implementation is anticipated during the April 1985 outage. The average drywell temperature (Technical Specification 3.6.1.7) has not exceeded 135°F during any event. This shows the heating problems are only localized conditions caused by stratified hot air.

Immediate Corrective Action

Primary Containment Cooling System (CRA) temperature controllers were adjusted and different combinations of CRA fan units were tried during the events. Cooling system modification status, Plant status and system time response contributed to the over temperature conditions existing for various times. In all events the temperatures were returned to allowable values.

Further Corrective Action

The Equipment Qualification Engineering group will address the long term significance of these overtemperature events.

Modifications to the design of the Drywell Cooling System have been made. Further modifications are planned for implementation during the planned April 1985 outage.

A final report on modifications, results achieved and long term significance will be provided at the conclusion of these modifications.

Safety Significance

There are a limited number of Class I components in the areas of concern. The time and size of the over temperature conditions have been determined to have negligible short term effects when compared to the temperature aging effects experienced over a normal operating lifetime. These localized temperature effects have not jeopardized Plant safety or that of the public. At no time was the Technical Specification allowed maximum temperature of 180°F exceeded.

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

November 29, 1984

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 84-034-04

Dear Sir:

Transmitted herewith is Licensee Event Report No. 84-034-04 for WNP-2 Plant. This report is a special report and is submitted in accordance with the requirements of the WNP-2 Technical Specification Section 6.9.2 and provides supplemental information to LER 84-034-03.

Very truly yours,

JM Powers for

J. D. Martin (M/D 927M)
WNP-2 Plant Manager

JDM:DK:mm

Enclosure:
Licensee Event Report No. 84-034-04

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

IE22
1/1